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DMB-016

Docket No. 50-366

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Dear Mr. Beckham:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated December 21, 1983, as supplemented April 16 and May 2, 1984.

The amendment revises the TSs for Hatch Unit 2 to: 1) increase the number of traveling incore probe (TIP) system detectors that are required to be operable from three to four, and 2) allow operation of the TIP system with one or more inoperable detectors.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Monthly Notice.

Sincerely,

"Authorized Signature BY
JOHN F. STOLZ"

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 43
2. Safety Evaluation

cc w/enclosures:
See next page

ORB#4:DL
RIngram
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ORB#4:DL
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GLainas
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P PDR

Hatch 1/2
Georgia Power Company

50-321/366

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-366
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated December 21, 1983, as supplemented April 16 and May 2, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

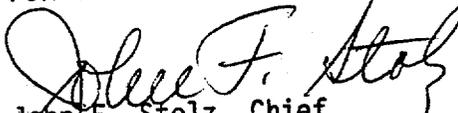
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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 43, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 31, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 43

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the areas of change.

Remove

3/4 3-57

Insert

3/4 3-57

INSTRUMENTATION

TRAVERSING INCORE PROBE SYSTEM

LIMITING CONDITION FOR OPERATION

3.3.6.6. The traversing incore probe system shall be OPERABLE with:

- a. Four movable detectors, drives and readout equipment to map the core, and
- b. Indexing equipment to allow all four detectors to be normalized in a common location.

APPLICABILITY:

When the traversing incore probe is used for:

- a. Recalibration of the LPRM detectors,
- b. Monitoring the APLHGR, LHGR, or MCPR, and
- c. Adjustment of the APRM setpoints.

ACTION:

With the traversing incore probe system inoperable preventing normalization of the TIP detectors, do not use the system for the above applicable monitoring or calibration functions for more than 31 EFPD following the last normalization. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.6.6 The traversing incore probe system shall be demonstrated OPERABLE by normalizing each of the above required detector outputs prior to or during use when required for the above applicable monitoring or calibration functions, if not performed within the previous 31 EFPD.

INSTRUMENTATION

CHLORINE DETECTORS

LIMITING CONDITION FOR OPERATION

3.3.6.7 Two independent chlorine detectors located in the main control room air intake, with their alarm/trip setpoints adjusted to actuate at a chlorine concentration of ≤ 5 ppm, shall be OPERABLE.

APPLICABILITY: CONDITIONS 1, 2, and 3.

ACTION:

- a. With one of the above required chlorine detectors inoperable, restore the inoperable detector to OPERABLE status within 7 days or, within the next 6 hours, initiate and maintain operation of the main control room environmental control system in the isolation mode of operation.
- b. With no chlorine detectors OPERABLE, within one hour initiate and maintain operation of the main control room environmental control systems in the isolation mode of operation.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.6.7 Each of the above required chlorine detectors shall be demonstrated OPERABLE by performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-366

1.0 Introduction

By letter dated December 21, 1983, Georgia Power Company (the licensee) requested an amendment to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit No. 2. This request was supplemented by additional information on April 16 and May 2, 1984. The proposed change to Technical Specification 3.3.6.6 would allow operation of the traversing incore probe (TIP) system for up to 31 effective full power days (EFPD) with one or more system probes in an inoperable condition. At present, the Technical Specifications prevent operation with properly functioning probes when more than one system probe is inoperable.

2.0 Evaluation

According to Technical Specification 3.3.6.6, the TIP system is presently used for the following functions:

- (a) recalibration of the local power range monitors (LPRM);
- (b) monitoring the average planar linear heat generation rate (APLHGR), the linear heat generation rate (LHGR), or the minimum critical power ratio (MCPR);
- (c) adjustment of the average power range monitor (APRM) setpoints.

Recalibration of the LPRM detectors requires usage of the OD-1 process computer program which can be run only if all four TIP machines are operable. Therefore, the proposed changes would have no effect on function (a).

When the plant process computer is operable, functions (b) and (c) above do not directly utilize TIP data. The proposed changes would, therefore, not affect these functions under normal conditions.

If the process computer and its backup are unavailable, however, the monitoring and setpoint adjustments of functions (b) and (c) could be performed manually with the P-1B calculation using TIP data. General Electric Company has noted in NEDO-25443 that the P-1B calculation can be performed acceptably with only a portion of the TIP system functioning. In these fairly rare instances, plant procedures and guidelines are designed to prevent usage of this manual calculation in a non-conservative manner.

Process computer program OD-2 is used to provide substitute or updated LPRM readings for a particular LPRM string. When this program is run, the new data obtained from the TIP system can be used to update the Core Maximum Fraction of Limiting Power Density (CMFLPD) thermal limit. If the CMFLPD exceeds the Fraction of Rated Thermal Power (F RTP), Technical Specifications require the APRM setpoints to be adjusted to correct the situation. Since APRM adjustment requires operability of the TIP system, the current Technical Specifications effectively prohibit use of the OD-2 calculation with less than three TIP probes operable. The proposed Technical Specification requirements would allow usage of OD-2 when only a portion of the TIP system is operable. The calibration constants for all LPRMs other than those in the traversed channel remain unaffected by use of the OD-2 program. In addition, OD-2 will not affect the results from P-1B since machine normalization factors and LPRM calibration constants are not involved in the P-1B calculation. During periods of degraded TIP system operability, usage of OD-2 would not permit operation of the reactor at a power level higher than is currently allowed.

The requested Technical Specification change increases the number of traversing incore probes from three to four. The increase in the number of probes required is a conservative change and constitutes an additional restriction on system operability.

Based on our review as described above, we find the requested revisions to Technical Specification 3.3.6.6 acceptable.

3.0 Environmental Considerations

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 31, 1985

Principal Contributor: L. Kopp