

August 27, 1985

Docket No. 50-366

DISTRIBUTION

Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
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Dear Mr. Beckham:

The Commission has issued the enclosed amendment to Facility Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated March 19, 1985.

The amendment revises the TSs to provide operating and surveillance requirements for automatic depressurization system bypass timers that were added to Unit 2 to satisfy the requirements of NUREG-0737 Item II.K.3.18.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Biweekly Notice.

Sincerely,

Original signed by

George W. Rivenbark, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 52
2. Safety Evaluation

cc w/enclosures:
See next page

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for
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GRivenbark;cr
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for
C-ORB#4:DL
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GLainas
8/21/85

Mr. J. T. Beckham, Jr.
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-366
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated March 19, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 52, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 27, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 52

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

3/4 3-27
3/4 3-29
3/4 3-32

Insert

3/4 3-27
3/4 3-29
3/4 3-32

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITIONS#</u>
3. HIGH PRESSURE COOLANT INJECTION SYSTEM		
a. Reactor Vessel Water Level - Low Low (Level 2) (2B21-N692 A,B,C,D)	2	1, 2, 3
b. Drywell Pressure - High (2E11-N694 A,B,C,D)	2	1, 2, 3
c. Condensate Storage Tank Level-Low (2E41-N002, 2E41-N003)	2(b) (c)	1, 2, 3
d. Suppression Chamber Water Level-High (2E41-N662B,D)	2(b) (c)	1, 2, 3
e. Logic Power Monitor (2E41-K1)	1(a)	1, 2, 3
f. Reactor Vessel Water Level-High (Level 8) (2B21-N693 B,D)	2	1, 2, 3
4. AUTOMATIC DEPRESSURIZATION SYSTEM		
a. Drywell Pressure - High (Permissive) (2E11-N694A,B,C,D)	2	1, 2, 3
b. Reactor Vessel Water Level - Low Low Low (Level 1) (2B21-N691 A,B,C,D)	2	1, 2, 3
c. ADS Timer (2B21-K5A,B)	1	1, 2, 3
d. ADS Low Water Level Actuation Timer (2B21-K754A,B; 2B21-K756A,B)	2	1, 2, 3
e. Reactor Vessel Water Level-Low (Level 3) (Permissive) (2B21-N695A,B)	1	1, 2, 3
f. Core Spray Pump Discharge Pressure - High (Permissive) (2E21-N655A,B; 2E21-N652A,B)	2	1, 2, 3
g. RHR (LPCI MODE) Pump Discharge Pressure - High (Permissive) (2E11-N655A,B,C,D; 2E11-N656A,B,C,D)	2/loop	1, 2, 3
h. Control Power Monitor (2B21-K1A,B)	1/bus(a)	1, 2, 3
5. LOW LOW SET S/RV SYSTEM		
a. Reactor Steam Dome Pressure - High (Permissive) (2B21-N620A,B,C,D)	2	1, 2, 3

(a) Alarm only. When inoperable, verify power availability to the bus at least once per 12 hours or declare the system inoperable.
 (b) Provides signal to HPCI pump suction valves only.
 (c) When either channel of the automatic transfer logic is inoperable, align HPCI pump suction to the suppression pool.
 # HPCI and ADS are not required to be OPERABLE with reactor steam dome pressure ≤ 150 psig.

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>CORE SPRAY SYSTEM</u>		
a. Reactor Vessel Water Level - Low Low Low (Level 1)	> -121.5 inches*	> -121.5 inches*
b. Drywell Pressure - High	< 1.85 psig	< 1.85 psig
c. Reactor Steam Dome Pressure - Low	> 422 psig**	> 422 psig**
d. Logic Power Monitor	NA	NA
2. <u>LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM</u>		
a. Drywell Pressure - High	< 1.85 psig	< 1.85 psig
b. Reactor Vessel Water Level - Low Low Low (Level 1)	> -121.5 inches*	> -121.5 inches*
c. Reactor Vessel Shroud Level (Level 0) - High	> -207 inches*	> -207 inches*
d. Reactor Steam Dome Pressure-Low	> 422 psig**	> 422 psig**
e. Reactor Steam Dome Pressure-Low	> 325 psig	> 325 psig
f. RHR Pump Start - Time Delay Relay		
1) Pumps A, B and D	10 ± 1 seconds	10 ± 1 seconds
2) Pump C	0.5 ± 0.5 seconds	0.5 ± 0.5 seconds
g. Logic Power Monitor	NA	NA

*See Bases Figure B 3/4 3-1.

**This trip function shall be less than or equal to 500 psig.

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
3. <u>HIGH PRESSURE COOLANT INJECTION SYSTEM</u>		
a. Reactor Vessel Water Level - Low Low (Level 2)	≥ -55 inches*	≥ -55 inches*
b. Drywell Pressure-High	≤ 1.85 psig	≤ 1.85 psig
c. Condensate Storage Tank Level - Low	≥ 0 inches**	≥ 0 inches**
d. Suppression Chamber Water Level - High	≤ 33.2 inches	≤ 33.2 inches
e. Logic Power Monitor	NA	NA
f. Reactor Vessel Water Level-High (Level 8) *	≤ 56.5 inches	≤ 56.5 inches
4. <u>AUTOMATIC DEPRESSURIZATION SYSTEM</u>		
a. Drywell Pressure-High	≤ 1.85 psig	≤ 1.85 psig
b. Reactor Vessel Water Level - Low Low Low (Level 1)	≥ -121.5 inches*	≥ -121.5 inches*
c. ADS Timer	≤ 120 seconds	≤ 120 seconds
d. ADS Low Water Level Actuation Timer	≤ 13 minutes	≤ 13 minutes
e. Reactor Vessel Water Level - Low (Level 3)	≥ 8.5 inches*	≥ 8.5 inches*
f. Core Spray Rump Discharge Pressure - High	≥ 130 psig	≥ 130 psig
g. RHR (LPCI MODE) Rump Discharge Pressure - High	≥ 105 psig	≥ 105 psig
h. Control Power Monitor	NA	NA
5. <u>LOW LOW SET S/RV SYSTEM</u>		
a. Reactor Steam Dome Pressure - High	≤ 1054 psig	≤ 1054 psig

*See Bases Figure B 3/4 3-1.

**Equivalent to 10,000 gallons of water in the CST.

TABLE 3.3.3-3

EMERGENCY CORE COOLING SYSTEM RESPONSE TIMES

<u>EGCS</u>	<u>RESPONSE TIME (Seconds)</u>
1. CORE SPRAY SYSTEM	≤ 27
2. LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM	≤ 40
3. HIGH PRESSURE COOLANT INJECTION SYSTEM	≤ 30
4. AUTOMATIC DEPRESSURIZATION SYSTEM	NA
5. ARM LOW LOW SET SYSTEM	≤ 1

TABLE 4.3.3-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
1. <u>CORE SPRAY SYSTEM</u>				
a. Reactor Vessel Water Level - Low Low Low (Level 1)	S	H	R	1, 2, 3, 4, 5
b. Drywell Pressure - High	S	H	R	1, 2, 3
c. Reactor Steam Dome Pressure - Low	S	H	R	1, 2, 3, 4, 5
d. Logic Power Monitor	NA	R	NA	1, 2, 3, 4, 5
2. <u>LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM</u>				
a. Drywell Pressure - High	S	H	R	1, 2, 3
b. Reactor Vessel Water Level - Low Low Low (Level 1)	S	H	R	1, 2, 3, 4*, 5*
c. Reactor Vessel Shroud Level (Level 0) - High	S	H	R	1, 2, 3, 4*, 5*
d. Reactor Steam Dome Pressure - Low	S	H	R	1, 2, 3, 4*, 5*
e. Reactor Steam Dome Pressure - Low	S	H	R	1, 2, 3, 4*, 5*
f. RHR Pump Start-Time Delay Relay	NA	NA	R	1, 2, 3, 4*, 5*
g. Logic Power Monitor	NA	R	NA	1, 2, 3, 4*, 5*

*Not applicable when two core spray system subsystems are OPERABLE per Specification 3.5.3.1.

TABLE 4.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED#</u>
3. <u>HIGH PRESSURE COOLANT INJECTION SYSTEM</u>				
a. Reactor Vessel Water Level - Low Low (Level 2)	S	M	R	1, 2, 3
b. Drywell Pressure-High	S	M	R	1, 2, 3
c. Condensate Storage Tank Level - Low	NA	M	Q	1, 2, 3
d. Suppression Chamber Water Level - High	S	M	R	1, 2, 3
e. Logic Power Monitor	NA	R	NA	1, 2, 3
f. Reactor Vessel Water Level - High (Level 8)	S	M	R	1, 2, 3
4. <u>AUTOMATIC DEPRESSURIZATION SYSTEM</u>				
a. Drywell Pressure - High	S	M	R	1, 2, 3
b. Reactor Vessel Water Level - Low Low Low (Level 1)	S	M	R	1, 2, 3
c. ADS Timer	NA	NA	R	1, 2, 3
d. ADS Low Water Level Actuation Timer	NA	NA	R	1, 2, 3
e. Reactor Vessel Water Level - Low (Level 3)	S	M	R	1, 2, 3
f. Core Spray Rump Discharge Pressure - High	S	M	R	1, 2, 3
g. RHR (LPCI MODE) Rump Discharge Pressure - High	S	M	R	1, 2, 3
h. Control Power Monitor	NA	R	NA	1, 2, 3
5. <u>LOW LOW SET S/RV SYSTEM</u>				
a. Reactor Steam Dome Pressure - High	S	M	R	1, 2, 3

#HPCI and ADS are not required to be OPERABLE with reactor steam dome pressure ≤ 150 psig.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-366

Evaluation

By letter dated March 19, 1985, Georgia Power Company (GPC) requested changes to the Technical Specifications (TSs). These changes reflect the modifications to the automatic depressurization system (ADS) logic, by the addition of limiting conditions of operation and surveillance requirements for the "bypass" timers.

For transient and accident events which do not directly produce a high drywell pressure signal and are degraded by a loss of all high pressure injection systems, manual actuation of the ADS is required to provide adequate core cooling. The ADS logic is modified such that the drywell pressure permissive in the current logic is bypassed after a sustained low reactor water level (Level 1). The ADS will initiate when both the bypass timer and the existing ADS timer have expired. The licensee will reconsider a manual ADS inhibit switch as part of the resolution of the ATWS issue by October 1985.

General Electric performed the plant specific analyses to determine the setting of the bypass timer for the ADS logic modification. The setting for the bypass timer is based on limiting the fuel peak cladding temperature of a transient or outside containment line break event to approximately 1500°F. The limiting event is the main steam line break outside the containment. General Electric Report, AE-52-0484, April 1984, presents the results of the analyses. The recommended setting for the bypass timer is 13 minutes and the proposed TS trip setpoint of < 13 minutes is consistent with this recommendation.

We have accepted this design modification as evaluated in our letters to GPC dated June 3, 1985 and June 5, 1985.

We find the proposed changes to the TSs acceptable.

ENVIRONMENTAL CONSIDERATIONS

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and

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changes in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 27, 1985

Principal Contributor: K. Desai