

January 30, 1987

Docket No. 50-366

Mr. James P. O'Reilly
Senior Vice President - Nuclear Operations
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

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Dear Mr. O'Reilly:

The Commission has issued the enclosed Amendment No. 73 to Facility Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit No. 2. In response to your request dated January 27, 1987, the amendment has been prepared and issued on an emergency basis in support of a waiver to a Technical Specification requirement granted orally on the night of January 26, 1987 in order to permit Unit 2 to restart following a forced outage that occurred earlier the same day. The waived requirement was one that prevents Unit 2 from changing modes of operation with the standby service water system inoperable. It consists of a change to the Unit 2 Technical Specifications adding a note that describes the one time waiver.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance and Final Determination of No Significant Hazards Consideration and Opportunity for Hearing will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by

George W. Rivenbark, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:

1. Amendment No. 73 to NPF-5
2. Safety Evaluation

cc w/enclosures:
See next page

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see note

DBL:PD#2
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Mr. J. P. O'Reilly
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated January 27, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

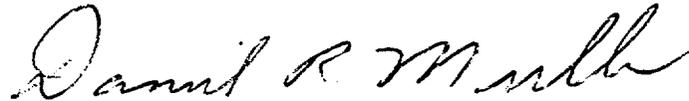
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment became effective January 26, 1987.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Daniel R. Muller".

Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 30, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines. The overleaf page is provided for convenience

Pages

3/4 7-3

PLANT SYSTEMS

SERVICE WATER SYSTEMS

LIMITING CONDITION FOR OPERATION

3.7.1.2 Two independent plant service water system loops and the standby service water subsystem shall be OPERABLE with:

- a. The river level at the intake structure \geq 61.7 feet Mean Sea Level USGS datum,
- b. Each plant service water system loop containing two OPERABLE plant service water pumps, and
- c. The standby service water system containing one OPERABLE standby service water pump.

APPLICABILITY: CONDITIONS 1, 2, 3, 4, and 5.

ACTION:

- a. In CONDITION 1, 2, or 3:
 1. With one plant service water pump inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable; restore the inoperable pump to OPERABLE status within 30 days or be in a least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 2. With one plant service water pump in each loop inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable; restore at least one inoperable pump to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 3. With one plant service water system loop inoperable, restore the inoperable loop to OPERABLE status with at least one OPERABLE pump within 72 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 4. With the standby service water subsystem inoperable* for up to 60 days, provide Hatch - Unit 1 service water cooling to the 1B Diesel generator by verifying OPERABILITY of the Hatch - Unit 1 service water cooling source per Hatch - Unit 1 technical specifications within 8 hours. Otherwise, declare the 1B diesel generator inoperable and take the ACTION required by Specification 3.8.1.1.

*The provisions of Specification 3.0.4 are not applicable for the startup from the January 26, 1987 outage.

PLANT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

b. In CONDITION 4 or 5:

1. With up to three plant service water pumps or one plant service water loop inoperable, or
2. With two plant service water pumps and the standby service water subsystem inoperable,

restore both plant service water loops with at least one pump in each loop and the standby service water subsystem to OPERABLE status within 7 days or declare the core spray system, the LPCI system and the associated diesel generators inoperable and take the ACTION required by Specifications 3.5.3.1, 3.5.3.2 and 3.8.1.2.

c. With the river level at the intake structure < 61.7 feet Mean Sea Level USGS datum, declare the plant service water system and the standby service water subsystem inoperable.

SURVEILLANCE REQUIREMENTS

4.7.1.2 The plant service water system and the standby service water subsystem shall be demonstrated OPERABLE:

- a. By verifying the river water level at the intake structure is ≥ 61.7 feet MSL;
 1. At least once per 14 days when the level is above 63 feet MSL, and
 2. At least once per 12 hours when the level is ≤ 63 feet MSL.
- b. At least once per 31 days by verifying that each valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed or otherwise secured in position, is in its correct position.
- c. At least once per 12 months by verifying the river bottom conditions in the vicinity of the intake structure.
- d. At least twice per 12 months by verifying the river stage discharge rating curve in the unit vicinity.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-366

1.0 INTRODUCTION

On January 26, 1987, while operating under Action Statement a.4 of Technical Specification (TS) 3.7.1.2, Unit 2 experienced a forced shutdown. The unit was operating under Action Statement a.4, because the Standby Service Water System (SSWS) was inoperable.

Once shutdown, the unit was prevented by TS 3.0.4 from restarting with the SSWS inoperable. TS 3.0.4 states that entry into an Operational Condition shall not be made unless the Limiting Conditions for Operation are met without reliance on provisions contained in Action Statements unless otherwise excepted.

By telephone conversations with the staff on January 26, 1987, Georgia Power Company (the licensee) requested a one-time exception to the provision of TS 3.0.4 as it relates to Action Statement a.4, on an emergency basis, to allow it to restart Unit 2. It stated that it was preparing a formal submittal requesting this emergency TS change. On the basis of these conversations and having determined that the unit could be safely restarted the staff orally granted a waiver from TS 3.0.4 as it applies to Action Statement a.4. while the request was being documented by the licensee and processed by the staff. By letter dated January 27, 1987 the licensee documented its request.

2.0 EVALUATION

TS 3.7.1.2 Action Statement a.4. requires the following upon the loss of the SSWS:

1. Align the Unit 1 Service Water System to cool the Standby Diesel Generator within 8 hours;

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2. Restore the SSWS to operable status within 60 days.

The alignment has been completed, however, TS 3.0.4 prevents the restart of Unit 2 while under the provisions to TS 3.7.1.2 Action Statement a.4. The one-time exception involves adding a statement that the provisions of TS 3.0.4 are not applicable to TS 3.7.1.2 Action Statement a.4 for the startup from the January 26, 1986 outage.

With the SSWS inoperable and the Standby Diesel Generator aligned to the Unit 1 Service Water System, no single active failure will result in the loss of cooling water to the Standby Diesel Generator. Since aligning the Unit 1 Service Water System to cool the Standby Diesel Generator maintains the availability of cooling, we find that allowing entry into a higher Operational Condition when the SSWS is inoperable has no impact on any of the assumptions or analyses of the Hatch Final Safety Analysis Report.

Based on the maintenance of cooling to the Standby Diesel Generator as discussed above, we conclude that removing, on a one-time basis, the applicability of TS 3.0.4 to the TS 3.7.1.2 Action Statement a.4 does not degrade the existing design bases and is acceptable.

3.0 EMERGENCY BASIS

The emergency results from the situation where Unit 2 could be safely started and operated with the SSWS inoperable, but was prevented by TS 3.0.4 from restarting following an earlier reactor trip unless relief from TS 3.0.4 was granted.

Unlike the other plant emergency diesel generators, Standby Diesel Generator 1B (which is shared between Hatch Unit 1 and 2) is normally supplied cooling water from an independent SSWS. When the SSWS is inoperable, the capability exists to provide an alternate safety-grade cooling water supply from the Unit 1 plant service water system. The subject SSWS was declared inoperable on January 7, 1987 due to inability to meet rated flow requirements and the cooling water supply was switched to the plant service water system. Inoperability of this system resulted in entry into a 60 day Limiting Condition for Operation (LCO) pursuant to TS 3.7.1.2 Action Statement a.4. While under this LCO, on January 26, 1987, Hatch Unit 2 experienced a forced outage unrelated to this condition. TS 3.0.4 prevents entry into different Operational Conditions (e.g. reactor restart) while under the provisions of TS Action Statements. This Specification thus prevents restart of Hatch Unit 2 from the shutdown of January 26, 1987, while under the provisions of TS 3.7.1.2 Action Statement a.4.

In telephone calls in the afternoon and evening of January 26, 1987 Georgia Power Company representatives informed the staff of the above information and stated that it was preparing an emergency request that TS 3.7.1.2 Action Statement a.4 be modified to include a statement that provisions of TS 3.0.4 are not applicable. Based on these telephone conversations it was determined by the Director, Division of Boiling Water Reactor Licensing that the plant could be safely restarted on a one-time basis and operated for the remainder of the 60 day LCO with the SSWS inoperable. The Director orally granted a waiver to the provision of TS 3.0.4 as it applies to the SSWS while the licensee prepared and submitted its written request for the emergency TS change and while the staff processed this request.

4.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that a license amendment involves no significant hazards consideration if operation of the facility in accordance with the amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

The requested change adds a note to TS 3.7.1.2 Action Statement a.4 that states that the provisions of TS 3.0.4 are not applicable during the restart from the January 26, 1986 outage. As noted above in this Safety Evaluation we have concluded that this change is acceptable.

Also as noted in this Safety Evaluation, the Standby Diesel Generator is aligned to the Unit 1 Service Water System for its cooling. This alignment maintains the availability of cooling over that provided by the SSWS. Further, allowing restart of Unit 2 with this cooling water alignment has no impact on any of the assumptions or analyses in the Unit 2 Final Safety Analysis Report.

On the basis, we have concluded that this TS change does not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

Accordingly, the staff has concluded that the amendment to Facility Operating License NPF-5 excepting TS TS 3.7.1.2 Action Statement a.4 from the provisions of TS 3.0.4 for startup from the January 26, 1987 outage involves no significant hazards consideration.

The State of Georgia was consulted on this matter and had no comments on the determination.

5.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final no significant hazards consideration finding with respect to the amendment. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Hearn

cooling water supply from the Unit 1 plant service water system. The

Dated: January 30, 1987