

September 9, 1985

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Docket No. 50-366

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Mr. J. T. Beckham, Jr.  
 Vice President - Nuclear Generation  
 Georgia Power Company  
 P. O. Box 4545  
 Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued Amendment No. 55 to Facility Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 15, 1985, as supplemented May 14, 1985.

The amendment revises the TSs to change the pressure alarm setpoint for the hydraulic control units and the identification number for the automatic depressurization system timer.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Biweekly Notice.

Sincerely,

**"ORIGINAL SIGNED BY"**

George W. Rivenbark, Project Manager  
 Operating Reactors Branch #4  
 Division of Licensing

Enclosures:

1. Amendment No. 55
2. Safety Evaluation

cc w/enclosures:  
 See next page

*for*  
 ORB#2:DL  
 SNorris  
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*GR*  
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*for*  
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*Rossard*  
 8/24/85

*G*  
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 GLainas  
 8/24/85

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Mr. J. T. Beckham, Jr.  
Georgia Power Company

Edwin I. Hatch Nuclear Plant,  
Units Nos. 1 and 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55  
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated February 15, 1985, as supplemented May 14, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

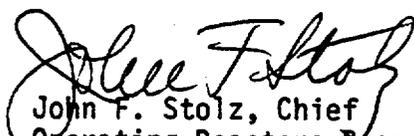
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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 55, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 9, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 55

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

3/4 1-8

3/4 3-27

Insert

3/4 1-8

3/4 3-27

## REACTIVITY CONTROL SYSTEMS

### FOUR CONTROL ROD GROUP SCRAM INSERTION TIMES

#### LIMITING CONDITION FOR OPERATION

---

3.1.3.4 The average scram insertion time, from the fully withdrawn position, for the three fastest control rods in each group of four control rods arranged in a two-by-two array, based on deenergization of the scram pilot valve solenoids as time zero, shall not exceed any of the following:

<u>Position Inserted From Fully Withdrawn</u>	<u>Average Scram Insertion Time</u>
46	0.379 seconds
36	1.162 seconds
26	1.972 seconds
6	3.624 seconds

APPLICABILITY: CONDITIONS 1 and 2.

#### ACTION:

With the average scram insertion times of the control rods exceeding the above limits, the provisions of Specification 3.0.4 are not applicable and operation may continue provided that:

- a. The control rods with the slower than average scram insertion times are declared inoperable,
- b. The requirements of Specification 3.1.3.1 are satisfied,
- c. An analysis is performed to determine that adequate scram reactivity remains for the slow rod combination, and
- d. The Surveillance Requirements of Specification 4.1.3.2.c are performed at least once per 60 days when operation is continued with three or more control rods with slow scram insertion times;

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.1.3.4 All control rods shall be scram time tested from the fully withdrawn position as required by Specification 4.1.3.2.

## REACTIVITY CONTROL SYSTEMS

### CONTROL ROD SCRAM ACCUMULATORS

#### LIMITING CONDITION FOR OPERATION

---

3.1.3.5 All control rod scram accumulators shall be OPERABLE.

APPLICABILITY: CONDITIONS 1,2 and 5\*

ACTION:

- a. In CONDITIONS 1 or 2 with one control rod scram accumulator inoperable, the provisions of Specification 3.0.4 are not applicable and operation may continue, provided that within 8 hours:
  1. The inoperable accumulator is restored to OPERABLE status, or
  2. The control rod associated with the inoperable accumulator is declared inoperable and the requirements of Specification 3.1.3.1 are satisfied.

Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.

- b. In CONDITION 5\* with a withdrawn control rod scram accumulator inoperable, fully insert the affected control rod and electrically disarm the directional control valves or close the withdraw isolation valve within one hour. The provisions of Specification 3.0.3 are not applicable.

#### SURVEILLANCE REQUIREMENTS

---

4.1.3.5 The control rod scram accumulators shall be determined OPERABLE:

- a. At least once per 7 days by verifying that the pressure and leak detectors are not in the alarmed condition, and
- b. At least once per 18 months by performance of a:
  1. CHANNEL FUNCTIONAL TEST of the leak detectors, and
  2. CHANNEL CALIBRATION of the pressure detectors to alarm at  $\geq$  940 psig.

\*At least the accumulator associated with each withdrawn control rod.  
Not applicable to control rods removed per Specification 3.9.11.1 or 3.9.11.2.

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITIONS#</u>
<b>3. HIGH PRESSURE COOLANT INJECTION SYSTEM</b>		
a. Reactor Vessel Water Level - Low Low (Level 2) (2B21-N692 A,B,C,D)	2	1, 2, 3
b. Drywell Pressure - High (2E11-N694 A,B,C,D)	2	1, 2, 3
c. Condensate Storage Tank Level-Low (2E41-N002, 2E41-N003)	2(b) (c)	1, 2, 3
d. Suppression Chamber Water Level-High (2E41-N662B,D)	2(b) (c)	1, 2, 3
e. Logic Power Monitor (2E41-K1)	1(a)	1, 2, 3
f. Reactor Vessel Water Level-High (Level 8) (2B21-N693 B,D)	2	1, 2, 3
<b>4. AUTOMATIC DEPRESSURIZATION SYSTEM</b>		
a. Drywell Pressure - High (Permissive) (2E11-N694A,B,C,D)	2	1, 2, 3
b. Reactor Vessel Water Level - Low Low Low (Level 1) (2B21-N691 A,B,C,D)	2	1, 2, 3
c. ADS Timer (2B21-K752 A, B)	1	1, 2, 3
d. ADS Low Water Level Actuation Timer (2B21-K754A,B; 2B21-K756A,B)	2	1, 2, 3
e. Reactor Vessel Water Level-Low (Level 3) (Permissive) (2B21-N695A,B)	1	1, 2, 3
f. Core Spray Rump Discharge Pressure - High (Permissive) (2E21-N655A,B; 2E21-N652A,B)	2	1, 2, 3
g. RHR (LPCI MODE) Rump Discharge Pressure - High (Permissive) (2E11-N655A,B,C,D; 2E11-N656A,B,C,D)	2/loop	1, 2, 3
h. Control Power Monitor (2B21-K1A,B)	1/bus(a)	1, 2, 3
<b>5. LOW LOW SET S/RV SYSTEM</b>		
a. Reactor Steam Dome Pressure - High (Permissive) (2B21-N620A,B,C,D)	2	1, 2, 3

- (a) Alarm only. When inoperable, verify power availability to the bus at least once per 12 hours or declare the system inoperable.
- (b) Provides signal to HPCI pump suction valves only.
- (c) When either channel of the automatic transfer logic is inoperable, align HPCI pump suction to the suppression pool.
- \* HPCI and ADS are not required to be OPERABLE with reactor steam dome pressure  $\leq 150$  psig.

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<b>1. CORE SPRAY SYSTEM</b>		
a. Reactor Vessel Water Level - Low Low Low (Level 1)	> -121.5 inches*	> -121.5 inches*
b. Drywell Pressure - High	< 1.85 psig	< 1.85 psig
c. Reactor Steam Dome Pressure - Low	> 422 psig**	> 422 psig**
d. Logic Power Monitor	NA	NA
<b>2. LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM</b>		
a. Drywell Pressure - High	< 1.85 psig	< 1.85 psig
b. Reactor Vessel Water Level - Low Low Low (Level 1)	> -121.5 inches*	> -121.5 inches*
c. Reactor Vessel Shroud Level (Level 0) - High	> -207 inches*	> -207 inches*
d. Reactor Steam Dome Pressure-Low	> 422 psig**	> 422 psig**
e. Reactor Steam Dome Pressure-Low	> 325 psig	> 325 psig
f. RHR Pump Start - Time Delay Relay	10 ± 1 seconds	10 ± 1 seconds
1) Pumps A, B and D	0.5 ± 0.5 seconds	0.5 ± 0.5 seconds
2) Pump C	NA	NA
g. Logic Power Monitor		

\*See Bases Figure B 3/4 3-1.

\*\*This trip function shall be less than or equal to 500 psig.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-366

Evaluation

By letter dated February 15, 1985, and supplemented by letter dated May 14, 1985, Georgia Power Company (GPC or the licensee) requested modifications of the Technical Specifications of the hydraulic control unit (HCU) to change the pressure alarm setpoint from  $955 \pm 15$  psig to  $\geq 940$  psig and the automatic depressurization system (ADS) timer relay nomenclature from (2B21-K5 A,B) to (2B21-K752 A,B).

The purpose of the HCU pressure alarms in the control room is to warn of a decreasing accumulator pressure condition which could result in insufficient stored energy in the accumulator to meet scram time requirements at low reactor pressure conditions. At normal operating conditions, reactor pressure alone is sufficient to insert a control rod.

The specification currently requires that the HCU pressure detectors be calibrated to alarm at  $955 \pm 15$  psig. Operating experience has shown that, due to temperature-induced instrument drift, this narrow tolerance cannot be met reasonably. It results in recurring failures of HCU to meet the operability requirements of the Technical Specifications. To eliminate this problem, the licensee has proposed to change the pressure alarm setpoint to  $\geq 940$  psig. The lower limit of the tolerance band is the important limit which is preserved by the channel calibration.

The pressure alarm limit of 940 psig will be unaffected by this change.

The ADS timer relay number is a nomenclature change and does not reflect any change in plant operation or setpoints.

We find the proposed changes to the TSs acceptable.

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### Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 9, 1985

Principal Contributor: K. Desai