September 11, 1985

DMB 016

Docket No. 50-366

DISTRIBUTION

Docket File	EJordan
NRC PDR	BGrimes
L PDR	WJones
ORB#4 Rdg	MVirgilio
HThompson	RDiggs
OELD	JPartlow
CMiles	RIngram
LHarmon	GRivenbark
ACRS-10	Gray File-4
TBarnhart-4	SECY

Mr. J. T. Beckham, Jr.
Vice President - Nuclear Generation
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Beckham:

The Commission has issued Amendment No. 57 to Facility Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit No. 2. In response to your request dated August 30, 1985, this amendment has been prepared and issued on an emergency basis to avoid the necessity of shutting down Unit 2. It consists of changes to the Technical Specification setpoint for automatic transfer of the reactor core isolation cooling pump suction from the condensate storage tank (CST) to the suppression pool on low CST water level.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance and Final Determination of No Significant Hazards Consideration and Opportunity for Hearing will be included in the Commission's next Biweekly Notice.

Sincerely,

Original signed by

George W. Rivenbark, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosure:
1. Amendment No. 57
2. Safety Evaluation

cc w/enclosures:
See next page

ORB#4:DL	ORB#4:DL	C-ORB#4:DL	OELD	AD:OR:DL
RIngram	GRivenbark;cf	JStolz		GLainas
9/9/85	9/6/85	9/6/85	9/9/85	9/11/85

See note 9/10/85

9/10/85 PM

8509170454 850911
PDR ADOCK 05000366
P PDR

Mr. J. T. Beckham, Jr.
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:
G. F. Trowbridge, Esq.
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, D.C. 20036

Mr. L. T. Gucwa
Engineering Department
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Mr. H. C. Nix, Jr., General Manager
Edwin I. Hatch Nuclear Plant
Georgia Power Company
P. O. Box 442
Baxley, Georgia 31513

Mr. Louis B. Long
Southern Company Services, Inc.
P. O. Box 2625
Birmingham, Alabama 35202

Resident Inspector
U.S. Nuclear Regulatory Commission
Route 1, P. O. Box 279
Baxley, Georgia 31513

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission,
101 Marietta Street, Suite 3100
Atlanta, Georiga 30303

Mr. Charles H. Badger
Office of Planning and Budget
Room 610
270 Washington Street, S.W.
Atlanta, Georgia 30334

Mr. J. Leonard Ledbetter, Commissioner
Department of Natural Resources
270 Washington Street, N.W.
Atlanta, Georgia 30334

Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) dated August 30, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

8509170455 850911
PDR ADOCK 05000366
P PDR

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 57 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 11, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 57

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove	Insert
3/4 3-35	3/4 3-35

TABLE 3.3.4-2

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION

FUNCTIONAL UNITS	TRIP SETPOINT	ALLOWABLE VALUE
a. Reactor Vessel Water Level - Low Low (Level 2)	≥ -55 inches*	≥ -55 inches*
b. Condensate Storage Tank Level - Low	≥ 0 inches**	≥ 0 inches**
c. Suppression Pool Water Level - High	≤ 151 inches	≤ 151 inches

*See Bases Figure B 3/4 3-1

**Equivalent to 10,000 gallons of water in the CST.

TABLE 4.3.4-1

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNITS	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION
a. Reactor Vessel Water Level- Low Low (Level 2)	S	M	R
b. Condensate Storage Tank Level- Low	NA	M	Q
c. Suppression Pool Water Level- High	NA	M	Q

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-366

Introduction

By letter dated August 30, 1985, Georgia Power Company (the licensee) requested a change in the Technical Specifications (TSs) for the Edwin I. Hatch Nuclear Plant, Unit 2. The change involves a correction of a footnote to the setpoint specification for the level in the condensate storage tank (CST) at which the suction of the reactor core isolation cooling system (RCIC) is automatically transferred to the suppression pool.

The licensee requested that this change be handled on an emergency basis because, on a technicality, the RCIC has been declared to be inoperable. Unless the system is restored to operable status within 14 days, the plant will be required to shutdown.

Evaluation

The change to the TSs involves a footnote to Table 3.3.4-2 "RCIC Actuation Instrumentation" on page 3/4 3-35. Item "b" in this table specifies the setpoint for the low level of the CST to be greater than or equal to zero inches. The reference footnote on this item states that zero inches correspond to 131 feet, 0 inches above mean sea level.

The licensee's proposal is to delete the reference to sea level and to replace it with a reference to 10,000 gallons of water in the CST.

TMI Action Plan Item II.K.3.22 required modification to BWR RCIC systems such that, like the high pressure core injection (HPCI) system, the suction source for RCIC would be transferred automatically from the CST to the suppression pool when low level is reached in the CST. The licensee installed level instruments in the CST at the same elevation as the HPCI instruments for the suction transfer of the RCIC. The associated TS proposed by the licensee in 1983 included an incorrect reference to the corresponding level above sea level, which read 131 feet, 0 inches. License Amendment No. 38 issued July 11, 1984, approved several TMI Action Plan items, including the incorrect cross-reference to sea level. The actual level is 130 feet, 10.5 inches. It was not recognized that the TS was incorrect until August 30, 1985.

8509170456 850911
PDR ADOCK 05000366
P PDR

The licensee has stated that the actual value of the actuation setpoint is correct and that the RCIC system is, and has been, capable of performing in the desired manner. The only problem is simply the incorrect reference to height above sea level. The licensee has stated also that, as a practical matter, the instrument could not be adjusted to the 131-foot value and provide reliable performance.

The licensee's proposal is that, rather than correct the reference value above sea level, a footnote should be changed to reference the remaining water volume in the tank (10,000 gallons). The 10,000 gallon value is part of the safety basis for selecting the low level setpoint and is therefore a more meaningful reference. Further, the corresponding footnote in the TS for the HPCI references the 10,000 gallon water volume, not elevation above sea level.

We have determined that the licensee's proposed change is, in fact, the correction of an error and, therefore, has no safety significance. In view of the information provided by the licensee and the considerations discussed above, the licensee's proposed change is acceptable. Approval of this permanent change on an emergency basis is also acceptable because it would be inappropriate to require a plant shutdown because of a technicality, when the system is capable of performing its intended safety function.

Emergency Circumstances

The emergency results from the licensee's discovery on August 30, 1985 that the trip setpoint for automatic transfer of the RCIC pump suction from the CST to the suppression pool on low CST water level is not consistent with the TS values for this setpoint. Upon discovering this discrepancy, the licensee, as required by the TS Action Statements, placed the Unit in a 14-day Limiting Condition for Operation (LCO). Unless this discrepancy is corrected before the 14-day LCO period expires, the plant will be required to shutdown. Since the trip setpoint cannot be adjusted to conform with the TS, the licensee has requested an emergency change to the TSs. The request for the emergency change to TSs was made on August 30, 1985, the date on which the discrepancy was discovered.

Final No Significant Hazards Consideration Determination

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that a license amendment involves no significant hazards consideration if operation of the facility in accordance with the amendment would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or

(2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or

(3) Involve a significant reduction in a margin of safety.

The requested change corrects an existing error in the TSs and results in a minor change in the trip setpoint for automatic transfer of the RCIC pump suction from the CST to the suppression pool on low CST water level. As noted above in this Safety Evaluation, we have concluded that this change is acceptable. The change does not affect the manner in which the plant is operated or the design bases for the plant.

Therefore, we conclude that:

(1) Operation of the facility in accordance with the amendment would not significantly increase the probability or consequences of an accident previously evaluated.

(2) Operation of the facility in accordance with the amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) Operation of the facility in accordance with the amendment would not involve a significant reduction in a margin of safety.

Accordingly, we conclude that the amendment to Facility Operating License NPF-5 revising the trip setpoint for automatic transfer of RCIC suction involves no significant hazards considerations.

The State of Georgia was consulted on this matter and had no comments on the determination.

Environmental Considerations

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final no significant hazards consideration finding with respect to this amendment. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 11, 1985

Principal Contributor: J. T. Beard