Dockets No.: 50-366

June 9, 1988

Mr. George F. Head
Senior Vice President
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Head:

SUBJECT: ISSUANCE OF AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NPF-5 - EDWIN I. HATCH NUCLEAR PLANT, UNIT 2 (TAC 66788)

The Commission has issued the enclosed Amendment No. 93 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit 2. The amendment consists of changes to the Technical Specifications in response to your application dated February 9, 1988.

The amendment revises the Technical Specifications to explain that the Reactor Water Cleanup System high differential flow isolation signal includes a 45-second time delay.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by:

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

Enclosures:
1. Amendment No. 93 to NPF-5
2. Safety Evaluation

cc w/ enclosures:
See next page

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4/15/88	4/15/88	6/8/88

Mr. George F. Head
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:
G. F. Trowbridge, Esq.
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N. W.
Washington, D.C. 20037

Mr. L. T. Gucwa
Engineering Department
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Nuclear Safety and Compliance Manager
Edwin I. Hatch Nuclear Plant
Georgia Power Company
P. O. Box 442
Baxley, Georgia 31513

Mr. Louis B. Long
Southern Company Services, Inc.
P. O. Box 2625
Birmingham, Alabama 35202

Resident Inspector
U.S. Nuclear Regulatory Commission
Route 1, Box 725
Baxley, Georgia 31513

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street, Suite 2900
Atlanta, Georiga 30323

Mr. Charles H. Badger
Office of Planning and Budget
Room 610
270 Washington Street, S.W.
Atlanta, Georgia 30334

Mr. J. Leonard Ledbetter, Commissioner
Department of Natural Resources
270 Washington Street, N.W.
Atlanta, Georgia 30334

Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513

DATED June 9, 1988

AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NPF-5, EDWIN I. HATCH, UNIT 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated February 9, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission.

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 93 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 9, 1988

OFFICIAL RECORD COPY

LA:PDII-3	PM:PDII-3	PSB	OGC-WF	PDII-3
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ATTACHMENT TO LICENSE AMENDMENT NO. 93

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Page	Insert Page
3/4 3-19	3/4 3-19
3/4 3-20	3/4 3-20

TABLE 3.3.2-3

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

TRIP FUNCTION	RESPONSE TIME (Seconds)#
1. PRIMARY CONTAINMENT ISOLATION	
a. Reactor Vessel Water Level	
1. Low (Level 3)	≤13*
2. Low Low (Level 2)	≤13*
3. Low Low Low (Level 1), except MSIVs	≤13*
b. Drywell Pressure - High	≤13*
c. Main Steam Line	
1. Radiation - High***	≤1.0**
2. Pressure - Low	≤13*
3. Flow - High	≤1.0**
4. Reactor Vessel Water Level - Low Low Low (Level 1)	≤1.0**
d. Main Steam Line Tunnel Temperature - High	≤13*
e. Condenser Vacuum - Low	NA
f. Turbine Building Area Temperature - High	NA
2. SECONDARY CONTAINMENT ISOLATION	
a. Reactor Building Exhaust Radiation - High***	≤13*
b. Drywell Pressure - High	≤13*
c. Reactor Vessel Water Level - Low Low (Level 2)	≤13*
d. Refueling Floor Exhaust Radiation - High***	≤13*

*The isolation actuation instrumentation response time shall be measured and recorded as a part of the ISOLATION SYSTEM RESPONSE TIME. Response time specified is diesel generator start delay time assumed in accident analysis.

**Isolation actuation instrumentation response time.

***Radiation detectors are exempt from response time testing. Response time shall be measured from detector output or the input of the first electronic component in the channel.

#Times to be added to valve movement times shown in Tables 3.6.3-1, 3.6.5.2-1 and 3.9.5.2-1 to obtain ISOLATION SYSTEM RESPONSE TIME for each valve.

##With time delay of 45 seconds.

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

TRIP FUNCTION	RESPONSE TIME (Seconds)#
3. REACTOR WATER CLEANUP SYSTEM ISOLATION	
a. Δ Flow - High	≤13*##
b. Area Temperature - High	≤13*
c. Area Ventilation Temperature ΔT - High	≤13*
d. SLCS Initiation	NA
e. Reactor Vessel Water Level-Low Low (Level 2)	≤13*
4. HIGH PRESSURE COOLANT INJECTION SYSTEM ISOLATION	
a. HPCI Steam Line Flow-High	3 ≤ Isolation Time ≤ 13*
b. HPCI Steam Supply Pressure - Low	≤13*
c. HPCI Turbine Exhaust Diaphragm Pressure - High	NA
d. HPCI Pipe Penetration Room Temperature - High	NA
e. Suppression Pool Area Ambient Temp. - High	NA
f. Suppression Pool Area ΔT - High	NA
g. Suppression Pool Area Temp. Timer Relays	NA
h. Emergency Area Cooler Temperature - High	NA
i. Drywell Pressure - High	≤13*
j. Logic Power Monitor	NA
5. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION	
a. RCIC Steam Line Flow - High	3 ≤ Isolation Time ≤ 13*
b. RCIC Steam Supply Pressure - Low	NA
c. RCIC Turbine Exhaust Diaphragm Pressure - High	NA
d. Emergency Area Cooler Temperature - High	NA
e. Suppression Pool Area Ambient Temp. - High	NA
f. Suppression Pool Area ΔT - High	NA
g. Suppression Pool Area Temperature Timer Relays	NA
h. Drywell Pressure - High	≤13*
i. Logic Power Monitor	NA
6. SHUTDOWN COOLING SYSTEM ISOLATION	
a. Reactor Vessel Water Level - Low (Level 3)	NA
b. Reactor Steam Dome Pressure - High	NA

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 93 TO

FACILITY OPERATING LICENSE NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

DOCKET NO. 50-366

1.0 INTRODUCTION

By letter dated February 9, 1988, Georgia Power Company (the licensee) requested a change to the Technical Specifications (TS) for Hatch Unit 2. The change would add a footnote to TS Table 3.3.2-3 to explain that the Reactor Water Cleanup (RWCU) system high differential flow isolation signal includes a 45-second time delay.

2.0 EVALUATION

Table 3.3.2-3 indicates a valve isolation response time of 13 seconds or less for the RWCU system upon receipt of a high differential flow signal. However, not stated is the fact that the isolation signal is not generated until the high differential flow condition has been present for 45 seconds. This delay is built in to avoid spurious signals that could be generated during startup and shutdown, instrument testing, pump start, or demineralizer change out.

The time delay has existed since Hatch Unit 2 was built and is a common design feature of BWRs. The BWR Standard Technical Specifications (NUREG-0123, Revision 3) acknowledge the existence of the delay timer:

Containment isolation for RWCU pipe breaks is provided by one of the following Class 1E circuits: Reactor Vessel Water level Low Low, RWCU Area Temperature High, or RWCU Area Ventilation Differential Temperature High. The RWCU high differential flow isolation is a single channel system provided for leak-before-break protection, but it is not relied upon in the accident analysis.

The change proposed by the licensee is thus a clarification of the design as actually exists at the plant. The change will not affect plant operation or plant safety in any manner. On this basis, the staff finds the proposed change acceptable.

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3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register on May 4, 1988 (53 FR 15911), and consulted with the state of Georgia. No public comments were received, and the state of Georgia did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lawrence P. Crocker, PDII-3/DRP-I/II

Dated: June 9, 1988

No significant hazards consideration comments received: No.

Local Public Document Room location: Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 9, 1988

Docket No. 50-366

MEMORANDUM FOR: Sholly Coordinator

FROM: Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II

SUBJECT: REQUEST FOR PUBLICATION IN BIWEEKLY FR NOTICE - NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE (TAC 66788)

Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, City of Dalton, Georgia, Docket No. 50-366, Edwin I. Hatch Nuclear Plant, Unit 2, Appling County, Georgia

Date of application for amendment: February 9, 1988

Brief description of amendment: The amendment revised the Technical Specifications to explain that the Reactor Water Cleanup System high differential flow isolation signal includes a 45-second time delay.

Date of issuance: June 9, 1988

Effective date: June 9, 1988

Amendment No.: 93

Facility Operating License No. NPF-5. Amendment revised the Technical Specifications.

Date of initial notice in FEDERAL REGISTER: May 4, 1988 (53 FR 15911)

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated June 9, 1988.

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