

February 25, 2002

Mr. Oliver D. Kingsley, President
and Chief Nuclear Officer
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1 (TMI-1) - REQUEST FOR
ADDITIONAL INFORMATION (TAC NO. MB2839)

Dear Mr. Kingsley:

In performing our review of your August 2, 2001, request for relief from certain requirements of Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code as required by Title 10 of the *Code of Federal Regulations*, Section 50.55a, the Nuclear Regulatory Commission (NRC) staff has determined that it will need the enclosed additional information to continue its review. These questions had previously been faxed to your staff and were discussed with your staff during a conference call on January 31, 2002. As a result of the conference call, the question faxed to your staff related to Relief Request RR-00-01 was rewritten to provide greater clarity, and the question related to Relief Request RR-00-06 was deleted. In order for the NRC staff to complete its review on schedule, please provide your response within 60 days receipt of this letter.

If you have any questions, please contact me at (301) 415-1402.

Sincerely,

/RA/

Timothy G. Colburn, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosure: Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION
THIRD 10-YEAR INTERVAL INSERVICE INSPECTION (ISI)
REQUESTS FOR RELIEF FOR
THREE MILE ISLAND NUCLEAR STATION, UNIT 1
DOCKET NUMBER: 50-289

RR-00-01

The 1995 Edition including the 1996 Addenda, of the ASME Code, Section XI, Appendix VIII addresses performance demonstration requirements for ultrasonic examination procedures, equipment, and personnel used to detect and size flaws. The licensee should address how it will meet the performance demonstration requirements of Appendix VIII for using ultrasonic examination techniques from the ID surface in detecting and sizing flaws on the OD surface.

Demonstrate that the techniques used for the B&W demonstration meet the objective of the Appendix VIII requirements providing assurance that surface flaws will be detected. What types/directions and size of flaws will the instrument, procedures and personnel be capable of detecting?

RR-00-02

Please describe your proposed VT-1 examination. Will the examination be a direct examination or a remote visual examination? What magnifications will be used? Describe the capabilities of your examination (such as resolution, detection and sizing capabilities). What percentage of the weld will the VT-1 examination cover (what percentage of the C-D surface area as shown in Figure IWB-2500-13 will be examined with this method)?

The NRC staff has also accepted UT examination from the accessible surface. (See safety evaluations for Brown's Ferry Nuclear Station, Units 2 and 3, 2nd 10-year ISI interval dated June 19, 2000, and Brown's Ferry Nuclear Station, Unit 2, 3rd 10-year interval, relief request 2-ISI-10, Revision 1, dated February 4, 2002.

RR-00-03

For the letdown coolers longitudinal welds, the Code requires the examination of 1 foot of longitudinal weld intersecting the circumferential weld?

What is the approximate length of the longitudinal weld that the licensee is able to examine? Why can't the licensee examine additional longitudinal welds and/or both coolers?

Provide a drawing(s) showing welds and identify accessible areas. What percentage of the required 1 foot are you able to obtain on one cooler?

What was examined in the first and second ISI intervals? Were there any unacceptable indications? What percentage of the required examination was attained?

Are there other examinations performed that help justify that the proposed inspection provides reasonable assurance of structural integrity?

Enclosure

REQUEST FOR ADDITIONAL INFORMATION
THIRD 10-YEAR INTERVAL INSERVICE INSPECTION (ISI)
REQUESTS FOR RELIEF FOR
THREE MILE ISLAND NUCLEAR STATION, UNIT 1
DOCKET NUMBER: 50-289

RR-00-05

How frequently are the decay heat removal coolers repaired/maintained so as to allow access to the weld location? Does the repair/maintenance program assure that one of the coolers will be disassembled each interval? Were either of the welds (DH-0399 and DH-0404) examined in the previous interval?

RR-00-07

Code Case N-546 is listed in the Draft Regulatory Guide (DG) 1091 which is out for public comment. The ADAMS accession number for DG-1091 is ML013120019. DG-1091 lists three conditions for the use of Code Case N-546. Please incorporate the three conditions. Should resolution of the public comments change the conditions the licensee will be expected to follow the requirements in the Code Case and any additional requirements listed in the future revision to Regulatory Guide 1.147.

RR-00-08

Initially the licensee references Class 1 and Class 2 under the heading of COMPONENT IDENTIFICATION. In the proposed alternative the licensee references only Class 1 systems. Will the alternative be applied to only Class 1 or to Class 1 and Class 2. (Code Case N-533-1 with a condition is also referenced in the Draft Regulatory Guide DG-1091)

RR-00-11

The alternative should be drafted in terms of Code Case-566-1 (Vogtle SER dated 5/4/2001)

RR-00-13

Rewrite in terms of CC N-598. (See SER for Vogtle dated 5/4/2001).
Licensee's proposed alternative did not address Table IWD.2412-1

Three Mile Island Nuclear Station, Unit No. 1

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Three Mile Island Nuclear Station, Unit No. 1

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