

April 28, 1989

Docket No.: 50-366

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NPF-5 -
EDWIN I. HATCH NUCLEAR PLANT, UNIT 2 (TAC 71999)

The Commission has issued the enclosed Amendment No. 99 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit 2. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated January 23, 1989.

The amendment changes the TS to allow Type C local leak rate tests to be conducted on the main steam isolation valves at intervals not to exceed two years and to specify the test pressure.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

LS

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 99 to NPF-5
2. Safety Evaluation

cc w/ enclosures:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 28, 1989

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Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
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Sincerely,

A handwritten signature in cursive script, reading "Lawrence P. Crocker", is written over a horizontal line.

Lawrence P. Crocker, Project Manager
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 99 to NPF-5
2. Safety Evaluation

cc w/ enclosures:
See next page

DATED April 28, 1989

AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NPF-5, EDWIN I. HATCH, UNIT 2

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M. Rood 14-H-25

L. Crocker 14-H-25

D. Hagan MNBB-3302

T. Meek (4) P1-137

W. Jones P-130A

ACRS (10) P-135

OGC-WF 15-B-18

ARM/LFMB AR-2015

GPA/PA 17-F-2

E. Butcher 11-F-23

L. Reyes RII

B. Grimes 9-A-2

E. Jordan MNBB-3302

J. Craig 8-D-1

Mr. W. G. Hairston, III
Georgia Power Company

Edwin I. Hatch Nuclear Plant,
Units Nos. 1 and 2

cc:

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Department of Natural Resources
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Chairman
Appling County Commissioners
County Courthouse
Baxley, Georgia 31513



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA
DOCKET NO. 50-366
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, (the licensee) dated January 23, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

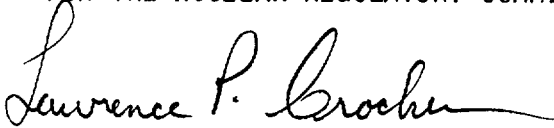
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 99, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


for David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 28, 1989

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 99, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

15/

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 28, 1989

OFFICIAL RECORD COPY

LA:PDII-3
MRood
3/28/89

PM:PDII-3
LCrocker:sw
3/28/89

DEST:SPLB
for Craig
3/28/89

OGC-WF
4/15/89

D:PDII-3
DBMatthews
4/24/89

ATTACHMENT TO LICENSE AMENDMENT NO. 99

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

Remove Page

3/4 6-5

Insert Page

3/4 6-5

SURVEILLANCE REQUIREMENTS (Continued)

- b.. If any periodic Type A test fails to meet either $.75 L_a$, or $.75 L_t$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet either $.75 L_a$ or $.75 L_t$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet either $.75 L_a$ or $.75 L_t$, at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 - 1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within $0.25 L_a$ or $0.25 L_t$,
 - 2. Has a duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test, and
 - 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage at P_a , 57.5 psig, or P_t , 28.8 psig.
- d. Type B and C tests* shall be conducted at P_a , 57.5 psig, during each reactor shutdown for refueling but in no case at intervals greater than 2 years except for tests involving:
 - 1. Air locks, which shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3, and
 - 2. Main steam line isolation valves, which shall be leak tested at 28.8 psig.
- e. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.
- f. The provisions of Specification 4.0.2 are not applicable.

*All Type B and Type C Leakage Tests (i.e., Local Leak Rate Tests) that fail (i.e., test leakage is such that an LER would be required) during an outage shall be reported according to 10 CFR 50.73 by one 30-day written report that is due within 30 days of the first leakage test failure in the outage. All other leakage test failures discovered during the outage will be reported in a revision to the original report due within 30 days after the end of the outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 99 TO

FACILITY OPERATING LICENSE NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

DOCKET NO. 50-366

1.0 INTRODUCTION

By letter dated January 23, 1989, Georgia Power Company (the licensee) requested a change to the Technical Specifications (TS) for the Edwin I. Hatch Nuclear Plant, Unit 2. The proposed change would allow Type C Local Leak Rate Tests (LLRTs) to be conducted on the main steam isolation valves (MSIVs) at intervals not to exceed two years rather than once per 18 months as presently required, and would specify the test pressure.

2.0 EVALUATION

Hatch Unit 2 TS 4.6.1.2.d.2 presently require a Type C leak test on MSIVs at least once per 18 months. In the past, this requirement has been met by testing the MSIVs during refueling outages. However, beginning with core loading for cycle 8, which is now underway, the licensee is moving toward an 18-month fuel cycle. Allowing for periods of operation at reduced power and for unscheduled outages, the actual time between refueling outages for Unit 2 is now expected to run longer than 18 months. The existing TS thus could result in a requirement for a special plant shutdown merely to conduct the MSIV tests.

10 CFR 50, Appendix J, Section III.D.3 requires that, "Type C tests shall be performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years." The licensee proposes to change TS 4.6.1.2.d to incorporate the wording of the regulation, thereby allowing the Type C tests on the MSIVs to be conducted at 2-year intervals. Since the testing of the MSIVs would be conducted in accordance with the requirements of the regulation, this change is acceptable.

TS 3.6.1.2.c specifies a pressure of 28.8 psig for the MSIV leak test. The licensee proposes to change TS 4.6.1.2.d.2 to state the 28.8 psig test pressure. This amendment clarifies the surveillance requirement, but does not change the actual conduct of the test. It is merely an administrative clarification and is, therefore, acceptable.

On the basis of the above, we find that the TS change proposed by the licensee will result in MSIV tests conducted in accordance with regulatory requirements and at the test pressure now specified elsewhere in the TS. The change is, therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register on March 8, 1989 (54 FR 9917), and consulted with the state of Georgia. No public comments were received, and the state of Georgia did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lawrence P. Crocker, PDII-3/DRP-I/II

Dated: April 28, 1989