

December 29, 1989

Docket No.: 50-366

Mr. W. G. Hairston, III  
Senior Vice President -  
Nuclear Operations  
Georgia Power Company  
P. O. Box 1295  
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: ISSUANCE OF AMENDMENT NO. 104 TO FACILITY OPERATING LICENSE NPF-5 -  
EDWIN I. HATCH NUCLEAR PLANT, UNIT 2 (TAC 74856)

The Commission has issued the enclosed Amendment No. 104 to Facility Operating License NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated August 22, 1989.

The amendment corrects two pages of the Unit 2 TSs to incorporate changes to the Reactor Protection System instrumentation surveillance requirements that previously were approved by Amendment 100 to the TSs but which were not properly incorporated in the TS pages issued with Amendment 100.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register Notice.

Sincerely,

Original Signed By:  
Lawrence P. Crocker, Project Manager  
Project Directorate II-3  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 104 to NPF-5
- 2. Safety Evaluation

cc w/ enclosures:  
See next page

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Mr. W. G. Hairston, III  
Georgia Power Company

Edwin I. Hatch Nuclear Plant,  
Units Nos. 1 and 2

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DATED December 29, 1989

AMENDMENT NO. 104 TO FACILITY OPERATING LICENSE NPF-5, EDWIN I. HATCH, UNIT 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA  
DOCKET NO. 50-366  
EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 104  
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit 2 (the facility) Facility Operating License No. NPF-5 filed by Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensee) dated August 22, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 104, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 29, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 104

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages

3/4 3-7  
3/4 3-41  
3/4 3-42

Insert Pages

3/4 3-7  
3/4 3-41  
3/4 3-42\*

\*Overleaf page provided to maintain document completeness.

TABLE 4.3.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION<sup>(a)</sup></u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
1. Intermediate Range Monitors:				
a. Neutron Flux - High	D	S/U <sup>(b)(c)</sup>	R	2
b. Inoperative	D NA	W W	R NA	3, 4, 5 2, 3, 4, 5
2. Average Power Range Monitor:				
a. Neutron Flux - Upscale, 15%	S	S/U <sup>(b)(c)</sup> , W <sup>(d)</sup>	S/U <sup>(b)</sup> , W <sup>(d)</sup>	2
b. Flow Referenced Simulated Thermal Power - Upscale	S	W S/U <sup>(b)</sup> , Q	W W <sup>(e)(f)</sup> , SA	5 1
c. Fixed Neutron Flux - Upscale, 118%	S	S/U <sup>(b)</sup> , Q	W <sup>(e)</sup> , SA	1
d. Inoperative	NA	Q	NA	1, 2, 5
e. Downscale	NA	W	NA	1
f. LPRM	D	NA	NA <sup>(g)</sup>	1, 2, 5
3. Reactor Vessel Steam Dome Pressure - High	S	Q	R	1, 2
4. Reactor Vessel Water Level - Low (Level 3)	S	Q	R	1, 2
5. Main Steam Line Isolation Valve - Closure	NA	Q	R	1
6. Main Steam Line Radiation - High	D	Q <sup>(i)</sup>	R	1, 2
7. Drywell Pressure - High	S	Q	R	1, 2
8. Scram Discharge Volume Water Level - High	NA	Q	R <sup>(h)</sup>	1, 2, 5

TABLE 4.3.5-1  
CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION(a)</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
1. APRM:				
a. Flow Referenced Simulated Thermal Power-Upscale	NA	S/U <sup>(b)</sup> , Q	R	1
b. Inoperative	NA	S/U <sup>(b)</sup> , Q	NA	1, 2, 5
c. Downscale	NA	S/U <sup>(b)</sup> , M	R	1
d. Neutron Flux - High, 12%	NA	S/U <sup>(b)</sup> , Q	R	2, 5
2. Rod Block Monitor:				
a. Upscale	NA	S/U <sup>(b)</sup> , Q	R	1 <sup>(d)</sup>
b. Inoperative	NA	S/U <sup>(b)</sup> , Q	NA	1 <sup>(d)</sup>
c. Downscale	NA	S/U <sup>(b)</sup> , Q	R	1 <sup>(d)</sup>
3. Source Range Monitors:				
a. Detector not full in	NA	S/U <sup>(b)</sup> , W	NA	2, 5
b. Upscale	NA	S/U <sup>(b)</sup> , W	R	2, 5
c. Inoperative	NA	S/U <sup>(b)</sup> , W	NA	2, 5
d. Downscale	NA	S/U <sup>(b)</sup> , W	R	2, 5
4. Intermediate Range Monitors:				
a. Detector not full in	NA	S/U <sup>(b)</sup> , W <sup>(c)</sup>	NA	2, 5
b. Upscale	NA	S/U <sup>(b)</sup> , W <sup>(c)</sup>	R	2, 5
c. Inoperative	NA	S/U <sup>(b)</sup> , W <sup>(c)</sup>	NA	2, 5
d. Downscale	NA	S/U <sup>(b)</sup> , W <sup>(c)</sup>	R	2, 5
5. Scram Discharge Volume:				
a. Water Level-High	NA	Q	R	1, 2, 5 <sup>(*)</sup>

TABLE 4.3.5-1 (Continued)

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SURVEILLANCE REQUIREMENTS

NOTES:

- a. Neutron detectors may be excluded from CHANNEL CALIBRATION.
- b. Within 24 hours prior to startup, if not performed within the previous 7 days.
- c. When changing from CONDITION 1 to CONDITION 2, perform the required surveillance within 12 hours after entering CONDITION 2.
- d. When THERMAL POWER exceeds the preset power level of the RWM and RSCS. The additional surveillance defined in Specification 4.1.4.3 will be required when the Limiting Condition defined in Specification 3.1.4.3 exists.
- e. With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.11.1 or 3.9.11.2.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 104 TO

FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY  
OGLETHORPE POWER CORPORATION  
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA  
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT 2

DOCKET NO. 50-366

1.0 INTRODUCTION

By letter dated August 22, 1989, Georgia Power Company (the licensee) requested changes to the Technical Specifications (TSs) for the Edwin I. Hatch Nuclear Plant, Unit 2. The proposed changes would conform TS Tables 4.3.1-1 and 4.3.5-1 to changes that previously were approved by Amendment 100 to the Unit 2 license.

2.0 EVALUATION

On June 6, 1989, Amendment 100 to the Unit 2 license was issued in response to a licensee change request dated March 27, 1986. The amendment changed the Reactor Protection System functional test frequency from monthly to quarterly and changed certain allowed equipment outage times.

Revised TS Table 4.3.1-1, issued as a part of Amendment 100, erroneously added a reference to a footnote "h" to the entry for the Main Steam Line Isolation Valve closure channel calibration frequency. However, footnote "h" applies only to the scram discharge volume water level-high calibration frequency. The requested change would delete the erroneous reference to footnote "h" from the entry for the Main Steam Line Isolation Valve, thereby correcting the table. It is, therefore, acceptable.

Amendment 100 changed the functional test frequencies for the Average Power Range Monitor (APRM) and the Rod Block Monitor instrumentation (except APRM downscale) from monthly to quarterly. However, these changes were not incorporated in TS Table 4.3.5-1. The requested change would modify Table 4.3.5-1 to incorporate the changes previously approved by Amendment 100. The change is, therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that

may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register, and consulted with the State of Georgia. No public comments were received, and the State of Georgia did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lawrence P. Crocker, PDII-3/DRP-I/II

Dated: December 29, 1989