

April 25, 1989

Docket Nos. 50-325
and 50-324

DISTRIBUTION
See attached list

Mr. Lynn W. Eury
Executive Vice President
Power Supply
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Eury:

SUBJECT: ISSUANCE OF AMENDMENT NO. 128 TO FACILITY OPERATING LICENSE
NO. DPR-71 AND AMENDMENT NO. 158 TO FACILITY OPERATING LICENSE NO.
DPR-62 - BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2,
REGARDING PRIMARY CONTAINMENT AIR LOCKS
(TAC NOS. 57054 AND 57055)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 128 to Facility Operating License No. DPR-71 and Amendment No. 158 to Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal dated October 24, 1984, as supplemented February 27, 1985, July 8, 1985 and March 17, 1987.

The amendments change the limiting condition for operation (LCO) and surveillance requirements for TS 3/4.6.1.3, Primary Containment Air Locks, to address the air lock door interlocks specifically. Additionally, the Technical Specifications (TS) for air locks would be reformatted to follow more closely the guidance of NUREG-0123, Standard Technical Specifications.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

E. G. Tourigny, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 128 to License No. DPR-71
2. Amendment No. 158 to License No. DPR-62
3. Safety Evaluation

[BSEP AMEND 57054/55]

cc w/enclosures:
See next page

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NAME	: PAnderson : E. G. Tourigny : jw : EAdensam	:	:	:	:	:	:	:
DATE	:04/19/89 :04/15/89 :04/24/89	:	:	:	:	:	:	:

Mr. Lynn W. Eury
Carolina Power & Light Company

Brunswick Steam Electric Plant
Units 1 and 2

cc:

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AMENDMENT NO. 128 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK, UNIT 1
AMENDMENT NO. 158 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK, UNIT 2

Docket File

~~NBC PDR~~
Local PDR
PDII-1 Reading
S. Varga, 14E4
G. Lainas 14H3
E. Adensam
P. Anderson
E. Tourigny
N. Le
L. Spessard (MNBB 3701)
OGC
D. Hagan (MNBB 3302)
E. Jordan (MNBB 3302)
B. Grimes (9A2)
T. Meeks (8) (P1-137)
W. Jones (P-130A)
E. Butcher (11F23)
H. Whitener, RII
ACRS (10)
GPA/PA
ARM/LFMB

DFOI
||

cc: Licensee/Applicant Service List



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 128
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated October 24, 1984, as supplemented February 27, 1985, July 8, 1985 and March 17, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 128, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

- 3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1989

*SEE PREVIOUS CONCURRENCE

OFC	:LA:PD21:DRPR*PM:PD21:DRPR*	OGC*	:D:PD21:DRPR	:	:	:
NAME	:PAnderson	:E:ourigny:jw:SHLewis	:EAdensam	:	:	:
DATE	:03/21/89	:03/28/89	:03/30/89	:04/25/89	:	:

4/19/89
4/19/89
OFFICIAL RECORD COPY

ATTACHMENT TO LICENSE AMENDMENT NO. 128

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 6-4
3/4 6-5
B 3/4 6-1

Insert Pages

3/4 6-4
3/4 6-5
B 3/4 6-1

CONTAINMENT SYSTEMS

PRIMARY CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 The primary containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of less than or equal to $0.05 L_a$ at P_a , 49 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2* and 3.

ACTION:

- a. With one primary containment air lock door inoperable:
 1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.
 2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- b. With the primary containment air lock door interlock inoperable:
 1. Lock the inner air lock door closed.
 2. Operation may then continue provided that the inner air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- c. With the primary containment air lock inoperable, except as a result of an inoperable air lock door or interlock, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

* See Special Test Exception 3.10.1.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each primary containment air lock shall be demonstrated OPERABLE:

- a. By verifying the seal leakage rate to be less than or equal to 5 scf per hour when the gap between the door seals is pressurized to 10 psig*:
 1. Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, and
 2. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when the air lock has been used and no maintenance has been performed on the air lock, and
 3. When the air lock seal has been replaced.
- b. By conducting an overall air lock leakage test at P_a , 49 psig, and by verifying that the overall air lock leakage is within its limit:
 1. At least once per six months[‡], and
 2. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when maintenance (except for seal replacement) has been performed on the air lock that would affect the air lock sealing capability.*
- c. By verification of air lock interlock OPERABILITY:
 1. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when the air lock has been used, and
 2. Prior to and following a drywell entry when PRIMARY CONTAINMENT INTEGRITY is required, and
 3. Following the performance of maintenance affecting the air lock interlock.

* Exemption to Appendix J of 10 CFR 50.

‡ The provisions of Specification 4.0.2 are not applicable.

3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 PRIMARY CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR Part 100 during accident conditions.

3/4.6.1.2 PRIMARY CONTAINMENT LEAKAGE

The limitations on primary containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure of 49 psig, P_a . As an added conservatism, the measured overall integrated leakage rate is further limited to less than or equal to $0.75 L_a$ or $0.75 L_t$, as applicable, during performance of the periodic tests to account for possible degradation of the containment leakage barriers between leakage tests.

Operating experience with the main steam line isolation valves has indicated that degradation has occasionally occurred in the leak tightness of the valves; therefore, the special requirement for testing these valves.

Exemptions from the requirements of 10 CFR Part 50 have been granted for main steam isolation valve leak testing, testing of airlocks after each opening, and leakage calculation methods.

Appendix J, paragraph III.A.3 requires that all Type A (Containment Integrated Leak Rate) tests be performed in accordance with ANSI N45.4-1972, "Leakage Rate Testing of Containment Structures for Nuclear Reactors." ANSI N45.4-1972 requires that leakage calculations be performed using the Point-to-Point or Total Time method. ANSI N45.4-1972 has been revised to a new standard, ANSI/ANS 56.8-1981, "Containment System Leakage Testing," which incorporates the Mass-Point method for leakage calculations. Type A tests will be performed in conformance with ANSI N45.4-1972 but will use the Mass-Point method for calculation of leakage rates as described in ANSI/ANS 56.8-1981.

3/4.6.1.3 PRIMARY CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on PRIMARY CONTAINMENT INTEGRITY and leak rate given in Specifications 3.6.1.1 and 3.6.1.2. The specification makes allowances for the fact that there may be long periods of time when the air locks will be in a closed and secured position during reactor operation. Only one closed door in each air lock is required to maintain the integrity of the containment. In the event of an inoperable door interlock, locking shut the inner door will ensure containment integrity while permitting access to the lock for maintenance and surveillance testing.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 158, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

- 3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1989

*SEE PREVIOUS CONCURRENCE

OFC	:LA:PD21:DRPR*PM:PD21:DRPR*	OGC*	:D:PD21:DRPR :	:	:
NAME	:PAnderson	:ETourigny:bd: SHLewis	:EAdersam	:	:
DATE	:03/21/89	:03/20/89	:03/30/89	:04/25/89	:

ATTACHMENT TO LICENSE AMENDMENT NO. 158

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 6-4
3/4 6-5
B 3/4 6-1

Insert Pages

3/4 6-4
3/4 6-5
B 3/4 6-1

CONTAINMENT SYSTEMS

PRIMARY CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 The primary containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of less than or equal to $0.05 L_a$ at P_a , 49 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2* and 3.

ACTION:

- a. With one primary containment air lock door inoperable:
 1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.
 2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- b. With the primary containment air lock door interlock inoperable:
 1. Lock the inner air lock door closed.
 2. Operation may then continue provided that the inner air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- c. With the primary containment air lock inoperable, except as a result of an inoperable air lock door or interlock, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

* See Special Test Exception 3.10.1.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each primary containment air lock shall be demonstrated OPERABLE:

- a. By verifying the seal leakage rate to be less than or equal to 5 scf per hour when the gap between the door seals is pressurized to 10 psig*:
 1. Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, and
 2. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when the air lock has been used and no maintenance has been performed on the air lock, and
 3. When the air lock seal has been replaced.
- b. By conducting an overall air lock leakage that at P_a , 49 psig, and by verifying that the overall air lock leakage is within its limit:
 1. At least once per six months[#], and
 2. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when maintenance (except for seal replacement) has been performed on the air lock that could affect the air lock sealing capability.*
- c. By verification of air lock interlock OPERABILITY:
 1. Prior to establishing PRIMARY CONTAINMENT INTEGRITY when the air lock has been used, and
 2. Prior to and following a drywell entry when PRIMARY CONTAINMENT INTEGRITY is required, and
 3. Following the performance of maintenance affecting the air lock interlock.

* Exemption of Appendix J of 10 CFR 50.

[#] The provisions of Specification 4.0.2 are not applicable

3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 PRIMARY CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR Part 100 during accident conditions.

3/4.6.1.2 PRIMARY CONTAINMENT LEAKAGE

The limitations on primary containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure of 49 psig, P_a . As an added conservatism, the measured overall integrated leakage rate is further limited to less than or equal to $0.75 L_a$ or $0.75 L_c$, as applicable, during performance of the periodic tests to account for possible degradation of the containment leakage barriers between leakage tests.

Operating experience with the main steam line isolation valves has indicated that degradation has occasionally occurred in the leak tightness of the valves; therefore, the special requirement for testing these valves.

Exemptions from the requirements of 10 CFR Part 50 have been granted for main steam isolation valve leak testing, testing of airlocks after each opening, and leakage calculation methods.

Appendix J, paragraph III.A.3 requires that all Type A (Containment Integrated Leak Rate) tests be performed in accordance with ANSI N45.4-1972, "Leakage Rate Testing of Containment Structures for Nuclear Reactors." ANSI N45.4-1972 requires that leakage calculations be performed using the Point-to-Point or Total Time method. ANSI N45.4-1972 has been revised to a new standard, ANSI/ANS 56.8-1981, "Containment System Leakage Testing," which incorporates the Mass-Point method for leakage calculations. Type A tests will be performed in conformance with ANSI N45.4-1972 but will use the Mass-Point method for calculation of leakage rates as described in ANSI/ANS 56.8-1981.

3/4.6.1.3 PRIMARY CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on PRIMARY CONTAINMENT INTEGRITY and leak rate given in Specifications 3.6.1.1 and 3.6.1.2. The specification makes allowances for the fact that there may be long periods of time when the air locks will be in a closed and secured position during reactor operation. Only one closed door in each air lock is required to maintain the integrity of the containment. In the event of an inoperable door interlock, locking shut the inner door will ensure containment integrity while permitting access to the lock for maintenance and surveillance testing.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 128 TO FACILITY OPERATING LICENSE NO. DPR-71
AND AMENDMENT NO. 158 TO FACILITY OPERATING LICENSE NO. DPR-62
CAROLINA POWER & LIGHT COMPANY et al.
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated October 24, 1984, as supplemented February 27, 1985, July 8, 1985 and March 17, 1987, Carolina Power & Light Company submitted a request for changes to the Brunswick Steam Electric Plant, Units 1 and 2.

The amendments change the limiting condition for operation (LCO) and surveillance requirements for TS 3/4.6.1.3, Primary Containment Air Locks, to address the air lock door interlocks specifically. Additionally, the Technical Specifications (TS) for air locks would be reformatted to follow more closely the guidance of NUREG-0123, Standard Technical Specifications.

2.0 EVALUATION

The licensee's request for a change to TS 3.6.1.3, Action Statements a and c, is consistent with the General Electric (GE) Standard Technical Specification 3.6.1.3. Action Statements a and c are acceptable. Action Statement 3.6.1.3.b of the licensee's submittal delineates the action to be taken in the event an inoperable air lock door interlock is identified.

Upon identification of an inoperable air lock door interlock, the actions required are as follows:

1. Lock the inner air lock door closed.
2. Continue plant operation provided the inner air lock door is verified to be locked closed at least once per 31 days.
3. Otherwise be in at least Hot Shutdown within the next 12 hours and Cold Shutdown within the following 24 hours.

The function of the air lock door interlock is to prevent the inadvertent opening of both the inner and outer air lock doors simultaneously while containment integrity is required. Locking the inner door closed and verifying the door remains locked closed is considered adequate protection against the simultaneous opening of both doors while permitting access to the air lock to perform leakage rate testing. Therefore, action statement b is acceptable.

The licensee also requested changes to the surveillance requirements. Surveillance a addresses the seal leakage rate test requirements. A new condition is added to perform the seal rate test (1) when the air lock seal has been replaced and (2) prior to establishing primary containment integrity when the air lock has been used and no maintenance has been performed in the air lock. This change is acceptable because it provides added assurance of the air lock sealing capability. Surveillance b addresses the overall air lock leakage test at 49 psig. A new condition has been added to perform this test prior to establishing containment integrity when maintenance (except for seal replacement) has been performed on the air lock that would affect the air lock sealing capability. This change is acceptable because it provides added assurance that primary containment integrity is in effect when needed.

Surveillance c is a new surveillance which addresses the air lock interlock operability. The surveillance provides adequate assurance that the interlock is operable and is acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

These amendments change a requirement with respect to installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released off site; and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that this amendment involves no significant hazards consideration which was published in the Federal Register originally on March 27, 1985 (50 FR 12139), and subsequently on August 27, 1985 (50 FR 34934), and September 23, 1987 (52 FR 35786), and consulted with the State of North Carolina. No public comments or requests for hearing were received, and the State of North Carolina did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: H. Whitener, RII
E. Tourigny, PD21

Dated: April 25, 1989