June 13, 1989

Docket Nos. 50-325
and 50-324

DISTRIBUTION
See attached list

Mr. Lynn W. Eury
Executive Vice President
Power Supply
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Eury:

SUBJECT: ISSUANCE OF AMENDMENT NO. 133 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2, REGARDING CHANGES FOR CONSISTENCY AND INDEX CORRECTION (TAC NOS. 69048 AND 69049)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 133 to Facility Operating License No. DPR-71 and Amendment No. 163 to Facility Operating License No. DPR-62, for Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal dated July 11, 1988, as supplemented April 11, 1989.

The amendments change the Technical Specifications to revise: (1) the titles listed in the index section of the Technical Specifications to be consistent with the titles shown in the text and (2) the two titles provided in the text, for consistency within the Technical Specifications between the two units. All changes are considered administrative changes.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original Signed By:

Ngoc B. Le, Project Manager
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 133 to License No. DPR-71
2. Amendment No. 163 to License No. DPR-62
3. Safety Evaluation

cc w/enclosures:
See next page

DF01
1/1

OFC	LA:PD21:DRPR	PM:PD21:DRPR	D:PD21:DRPR	OGC			
NAME	PAnderson	N Le	EAdensam				
DATE	5/16/89	5/16/89	5/16/89	5/19/89			

OFFICIAL RECORD COPY

8906220265 890613
PDR ADOCK 05000324
P PDC

CP

Mr. L. W. Eury
Carolina Power & Light Company

Brunswick Steam Electric Plant
Units 1 and 2

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AMENDMENT NO. 133 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK, UNIT 1
AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK, UNIT 2

Docket File
NRC PDR
Local PDR
PDII-1 Reading
G. Lainas (14E4)
E. Adensam (14H3)
E. Reeves
P. Anderson
E. Tourigny
N. Le
OGC
D. Hagan (MNBB 3302)
E. Jordan (MNBB 3302)
B. Grimes (9A2)
T. Meeks (8) (P1-137)
W. Jones (P-130A)
E. Butcher (11F23)
ACRS (10)
GPA/PA
ARM/LFMB

cc: Licensee/Applicant Service List

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 133
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated July 11, 1988, as supplemented April 11, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

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PDR ADOCK 05000324
P PDC

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 133 , are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 13, 1989

OFC	LA:PD21:DRPR	PM:PD21:DRPR	OGC	D:PD21:DRPR			
NAME	PAnderson	ETourigny		EAdensam			
DATE	4/ /89	5/10/89	5/25/89	5/16/89			

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LIQUID RADWASTE TREATMENT SYSTEM

LIMITING CONDITION FOR OPERATION

3.11.1.3 The liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to the liquid effluent from the site to UNRESTRICTED AREAS (see Figure 5.1.3-1) would exceed 0.12 mrem to the total body or 0.4 mrem to any organ in a 31-day period.

APPLICABILITY: At all times.

ACTION:

a. With radioactive liquid waste being discharged without treatment and in excess of the above limits, in lieu of a Licensee Event Report, prepare and submit to the Commission within 30 days, pursuant to Specification 6.9.2, a Special Report that includes the following information:

   1. Explanation of why liquid radwaste was being discharged without treatment, identification of any inoperable equipment or subsystem, and reason for the inoperability.

   2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and

   3. Summary of description of action(s) taken to prevent a recurrence.

b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.1.3 Doses due to liquid releases from the site to UNRESTRICTED AREAS shall be projected at least once per 31 days in accordance with the ODCM.

NOTE: See Bases 3/4.11.1.3

3/4.5.4 SUPPRESSION POOL

The OPERABILITY of the suppression pool in CONDITIONS 1, 2, or 3 is required by Specification 3.6.2.2. The suppression pool, as part of the ECCS, ensures that a sufficient supply of water is available in the event of a LOCA. This limit or suppression pool minimum water volume ensures that sufficient water is available to permit recirculation cooling flow to the core.

The technical specifications for cooling the reactor core do not make allowances for repair work that might require making the suppression pool inoperable unless all irradiated fuel is removed. This specification will permit those repairs to be made and at the same time give assurance that the irradiated fuel has an adequate cooling water supply when the suppression pool must be made inoperable, including draining.

6.5 REVIEW AND AUDIT

6.5.1 NUCLEAR SAFETY REVIEWERS

6.5.1.1 Individuals shall be designated/approved by the General Manager - Brunswick Plant for performing nuclear safety reviews.

6.5.1.2 Individuals designated under Specification 6.5.1.1 above shall have an academic degree in an engineering or related field or equivalent and two years related experience.

6.5.1.3 A list shall be maintained of individuals qualified to perform nuclear safety reviews, including additional individuals whose expertise may be necessary during the reviews to assure that the reviewers collectively possess the background and qualifications in the disciplines necessary and important to the specific review.

6.5.1.4 The list specified in Specification 6.5.1.3 above shall include the disciplines for which each individual is qualified.

6.5.1.5 For those cases where interdisciplinary reviews are required, as many individuals as necessary shall be used to perform the nuclear review function.

6.5.1.6 One of the two nuclear safety reviewers shall be an individual other than the original preparer of the individual approving the action.

6.5.2 SAFETY EVALUATIONS AND INDEPENDENT REVIEW CONTROL

SAFETY EVALUATIONS

6.5.2.1 A safety evaluation shall be prepared for each of the following:

a. Changes to procedures required by Specification 6.8, or changes to other procedures that affect nuclear safety.

b. Proposed tests or experiments that affect nuclear safety.

c. Proposed modifications to plant systems or equipment that affect nuclear safety.

d. Proposed changes to the Technical Specifications.

e. Proposed changes to the Operating License.

6.5.2.2 Two nuclear safety reviews of the item and safety evaluation(s) prepared in accordance with Specification 6.5.2.1 above shall be performed prior to approval and implementation.

6.5.2.3 The item and associated safety evaluation(s) shall be examined in order to determine whether an interdisciplinary review is required in accordance with Specification 6.5.1.5 above.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated July 11, 1988, as supplemented April 11, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163 , are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 13, 1989

OFC	LA:PD21:DRPR	PM:PD21:DRPR	OGC	D:PD21:DRPR		
NAME	PAnderson	~~ELounsbury~~		EAdensam		
DATE	5/10/89	5/16/89	5/25/89	5/12/89		

OFFICIAL RECORD COPY

ATTACHMENT TO LICENSE AMENDMENT NO. 163

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages	Insert Pages
III	III
V	V
VI	VI
VII	VII
VIII	VIII
IX	IX
IXa	IXa
X	X
XI	XI
XII	XII
XIIa	XIIa
XIV	XIV
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CONTROL ROD DRIVE HOUSING SUPPORT

LIMITING CONDITION FOR OPERATION

3.1.3.8 The control rod drive housing support shall be in place when there is fuel in the reactor vessel.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With the control rod drive housing support not in place, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.1.3.8 The control rod drive housing support shall be inspected after reassembly and verified to be in place prior to start-up any time it has been disassembled or when maintenance has been performed in the control rod drive housing support area.

LIQUID RADWASTE TREATMENT SYSTEM

LIMITING CONDITION FOR OPERATION

3.11.1.3 The liquid radwaste treatment system shall be used to reduce the radioactive materials in liquid wastes prior to their discharge when the projected doses due to the liquid effluent from the site to UNRESTRICTED AREAS (see Figure 5.1.3-1) would exceed 0.12 mrem to the total body or 0.4 mrem to any organ in a 31-day period.

APPLICABILITY: At all times.

ACTION:

a. With radioactive liquid waste being discharged without treatment and in excess of the above limits, in lieu of a Licensee Event Report, prepare and submit to the Commission within 30 days, pursuant to Specification 6.9.2, a Special Report that includes the following information:

   1. Explanation of why liquid radwaste was being discharged without treatment, identification of any inoperable equipment or subsystem, and reason for the inoperability.

   2. Action(s) taken to restore the inoperable equipment to OPERABLE status, and

   3. Summary of description of action(s) taken to prevent a recurrence.

b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.1.3 Doses due to liquid releases from the site to UNRESTRICTED AREAS shall be projected at least once per 31 days in accordance with the ODCM.

NOTE: See Bases 3/4.11.1.3

3/4.5.4 SUPPRESSION POOL

The OPERABILITY of the suppression pool in CONDITIONS 1, 2, or 3 is required by Specification 3.6.2.2. The suppression pool, as part of the ECCS, ensures that a sufficient supply of water is available in the event of a LOCA. This limit on suppression pool minimum water volume ensures that sufficient water is available to permit recirculation cooling flow to the core.

The technical specifications for cooling the reactor core do not make allowances for repair work that might require making the suppression pool inoperable unless all irradiated fuel is removed. This specification will permit those repairs to be made and at the same time give assurance that the irradiated fuel has an adequate cooling water supply when the suppression pool must be made inoperable, including draining.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 133 TO FACILITY OPERATING LICENSE NO. DPR-71

AND AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NO. DPR-62

CAROLINA POWER & LIGHT COMPANY, et al.

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2

DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated July 11, 1988, as supplemented April 11, 1989, Carolina Power & Light Company submitted a request for changes to the Technical Specifications (TS) for Brunswick Steam Electric Plant, Units 1 and 2. The proposed changes would revise: (1) the titles listed in the index section of the TS to be consistent with the titles shown in the text, and (2) the two titles provided in the text for consistency within TS between the two units. All changes are considered administrative changes.

The April 11, 1989 submittal corrected TS pages to reflect amendments issued since the original July 11, 1988 application. Therefore, the later submittal was not renoticed and the no significant hazards consideration determination published in February 1, 1989, was not affected.

2.0 EVALUATION

In the July 11, 1988 submittal, the licensee stated that the index for the BSEP TS does not, in all cases, accurately represent the information provided in the text of the document. In some cases, the titles of the sections are inconsistent; in others, the page numbers are incorrect. The licensee further stated that the changes are purely administrative. There are no changes to the text of the TS. Only the index and two titles in the text are being changed for consistency. There are no modifications or design changes involved. The sole purpose of these requests was to provide consistency between the index section of the TS and the text of the document and to provide consistency between the TS of the two units.

In the April 11, 1989 supplemental submittal, the licensee stated that certain index section pages previously included in the July 11, 1988 request no longer require revision due to the issuance of other license amendments since the original submittal. In addition, several Unit 1

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TS index section pages, which were omitted from the original submittal, were included in the April 11, 1989 submittal. The staff reviewed the requests and agreed with the changes proposed by the licensee.

Therefore, based on the above discussion, the NRC staff finds the changes to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment relates to changes in recordkeeping, or administrative procedures to show the index in all cases to accurately reflect the information provided in the text. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eliginility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepated in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that this amendment involves no significant hazards consideration which was published in the Federal Register (54 FR 5160) on February 1, 1989, and consulted with the State of North Carolina. No public comments or requests for hearing were received, and the State of North Carolina did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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