

July 13, 1989

Docket Nos. 50-325
and 50-324

DISTRIBUTION
See attached list

Mr. Lynn W. Eury
Executive Vice President
Power Supply
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Eury:

SUBJECT: ISSUANCE OF AMENDMENT NO. 135 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 165 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2, REGARDING DELETION OF THE RESIDUAL HEAT REMOVAL/SERVICE WATER DISCHARGE DIFFERENTIAL PRESSURE INSTRUMENT FROM TECHNICAL SPECIFICATIONS (TAC NOS. 68982 AND 68983)

The Nuclear Regulatory Commission has issued the enclosed Amendment No.135 to Facility Operating License No. DPR-71 and Amendment No. 165 to Facility Operating License No. DPR-62, for Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your submittal dated June 22, 1988.

The amendments change the TS to delete the residual heat removal (RHR)/service water discharge differential pressure instrument (transmitter and indicator) from the TS for each unit.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/s/

E. G. Tourigny, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 135 to License No. DPR-71
2. Amendment No. 165 to License No. DPR-62
3. Safety Evaluation

cc w/enclosures:
See next page

[BSEP AMEND 68982/68983]	
LA ADII-1	PM2DII-1
PAnderson	ETourigny:bd
06/14/89	06/17/89

Good
D:PDII-1
EAdensam
06/17/89

cc
[Handwritten signature]
[Handwritten initials]

8907200236 890713
PDR ADDCK 05000324
P
PDC

Mr. L. W. Eury
Carolina Power & Light Company

Brunswick Steam Electric Plant
Units 1 and 2

cc:

Mr. Russell B. Starkey, Jr.
Project Manager
Brunswick Nuclear Project
P. O. Box 10429
Southport, North Carolina 28461

Mr. H. A. Cole
Special Deputy Attorney General
State of North Carolina
P. O. Box 629
Raleigh, North Carolina 27602

Mr. R. E. Jones, General Counsel
Carolina Power & Light Company
P. O. Box 1551
Raleigh, North Carolina 27602

Mr. Robert P. Gruber
Executive Director
Public Staff - NCUC
P. O. Box 29520
Raleigh, North Carolina 27626-0520

Ms. Grace Beasley
Board of Commissioners
P. O. Box 249
Bolivia, North Carolina 28422

Resident Inspector
U. S. Nuclear Regulatory Commission
Star Route 1
P. O. Box 208
Southport, North Carolina 28461

Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta Street, Suite 2900
Atlanta, Georgia 30323

Mr. Dayne H. Brown, Chief
Radiation Protection Branch
Division of Facility Services
N. C. Department of Human Resources
701 Barbour Drive
Raleigh, North Carolina 27603-2008

Mr. J. L. Harness
Plant General Manager
Brunswick Steam Electric Plant
P. O. Box 10429
Southport, North Carolina 28461

AMENDMENT NO. 135 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK, UNIT 1
AMENDMENT NO. 165 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK, UNIT 2

Docket File

NRC PDR

Local PDR

PDII-1 Reading

S. Varga (14E4)

G. Lainas

E. Adensam

P. Anderson

E. Tourigny

N. Le

L. Spessard (MNBB 3701)

OGC

D. Hagan (MNBB 3302)

E. Jordan (MNBB 3302)

B. Grimes (9A2)

T. Meeks (8) (P1-137)

W. Jones (P-130A)

J. Cavo

J. Craig

S. Newberry

W. Hodges

ACRS (10)

GPA/PA

ARM/LFMB

cc: Licensee/Applicant Service List



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 135
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated June 22, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 135, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

- 3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 13, 1989

OFC	:LA:PD21:DRPR:PM:PD21:DRPR:	OGC	:D:PD21:DRPR	:SCSB	:RSB	:SPLB
NAME	:PAnderson	:E Courigny:bd:	:EAdensam	:SNewberry	:WHodges	:JCraig
DATE	:6/14/89	:6/21/89	:6/29/89	:7/1/89	:6/29/89	:6/26/89

ATTACHMENT TO LICENSE AMENDMENT NO. 135

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 3-57

3/4 3-58

Insert Pages

3/4 3-57

3/4 3-58

TABLE 3.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>READOUT LOCATION</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Vessel Pressure	RSP(a)	1
2. Reactor Vessel Water Level	RSP(a)	1
3. Suppression Chamber Water Level	RSP(a)	1
4. Suppression Chamber Water Temperature	RSP(a)	1
5. Drywell Pressure	RSP(a)	1
6. Drywell Temperature	RSP(a)	1
7. Residual Heat Removal Head Spray Flow	RSP(a)	1
8. Residual Heat Removal System Flow	RSP(a)	1

(a) Remote Shutdown Panel, Reactor Building 20' Elevation

TABLE 4.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure	M	Q
2. Reactor Vessel Water Level		
NO17 Instrument Loop:	NA	Q
Remaining Instruments:	M	Q
3. Suppression Chamber Water Level	M	R
4. Suppression Chamber Water Temperature	M	R
5. Drywell Pressure	M	Q
6. Drywell Temperature	M	R
7. Residual Heat Removal Head Spray Flow	M	Q
8. Residual Heat Removal System Flow	M	Q



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 165
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated June 22, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 165, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

- 3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 13, 1989

OFC	:LA:PD21:DRPR:PM:PD21:DRPR:	OGC	:D:PD21:DRPR	:SCSB	:RSB	:SPLB
NAME	:PAnderson	:ETofigny:bd:	:EAdensam	:SNewberry	:WHodges	:JCraig
DATE	:6/14/89	:6/21/89	:6/19/89	:6/12/89	:6/22/89	:6/26/89

ATTACHMENT TO LICENSE AMENDMENT NO. 165

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 3-57

3/4 3-58

Insert Pages

3/4 3-57

3/4 3-58

TABLE 3.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>READOUT LOCATION</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Vessel Pressure	RSP(a)	1
2. Reactor Vessel Water Level	RSP(a)	1
3. Suppression Chamber Water Level	RSP(a)	1
4. Suppression Chamber Water Temperature	RSP(a)	1
5. Drywell Pressure	RSP(a)	1
6. Drywell Temperature	RSP(a)	1
7. Residual Heat Removal Head Spray Flow	RSP(a)	1
8. Residual Heat Removal System Flow	RSP(a)	1

(a) Remote Shutdown Panel, Reactor Building 20' Elevation

TABLE 4.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure	M	Q
2. Reactor Vessel Water Level		
N017 Instrument Loop:	NA	Q
Remaining Instruments:	M	Q
3. Suppression Chamber Water Level	M	R
4. Suppression Chamber Water Temperature	M	R
5. Drywell Pressure	M	Q
6. Drywell Temperature	M	R
7. Residual Heat Removal Head Spray Flow	M	Q
8. Residual Heat Removal System Flow	M	Q



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 135 TO FACILITY OPERATING LICENSE NO. DPR-71
AND AMENDMENT NO. 165 TO FACILITY OPERATING LICENSE NO. DPR-62

CAROLINA POWER & LIGHT COMPANY, et al.

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2

DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated June 22, 1988, Carolina Power & Light Company submitted a request for changes to the Brunswick Steam Electric Plant, Units 1 and 2.

The amendments would delete the residual heat removal (RHR)/service water discharge differential pressure instrument (transmitter and indicator) from the Technical Specifications (TS) for each unit.

2.0 EVALUATION

Each Brunswick unit has a remote shutdown panel located in the reactor building at the twenty foot level. The panel is utilized to shut down the unit and maintain shutdown conditions in the event control room habitability is lost. TS are in place for each panel's monitoring instrumentation, which identifies operability requirements, action statements, and surveillances. One indication on each panel is differential pressure between the shell, or RHR side, and the tube, or service water side, of the RHR heat exchanger. The RHR heat exchanger serves as the heat sink for suppression pool cooling and shutdown cooling.

The differential pressure indication does not provide the operator any indication that the RHR and/or RHR service water system are operating or "in service." The instrumentation provides information on relative system pressures only; it does not enhance the operation of the RHR heat exchanger or RHR service water system with respect to establishing or maintaining hot or cold shutdown conditions.

The licensee's procedure entitled "Plant Shutdown from Outside Control Room" provides specific instructions for placing the RHR and RHR service water system into suppression pool cooling and shutdown cooling modes. Both modes of cooling are established and maintained by setting the RHR service water booster pump run currents and the RHR system flow to predetermined values. The differential pressure indication is not used

to establish or verify flow rates and reference to RHR heat exchanger tube-to-shell side pressure differential is not made in the procedure. The differential pressure indication does provide useful information such as relative pressure between systems, but this indication is not required for safe operation.

Based upon the above evaluation, the staff concludes that the subject differential pressure indication is not necessary, and the TS regarding the indication may be deleted. We find the amendment request acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

These amendments change a requirement with respect to installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released off site; and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that this amendment involves no significant hazards consideration which was published in the Federal Register (54 FR 23308) on May 31, 1989, and consulted with the State of North Carolina. No public comments or requests for hearing were received, and the State of North Carolina did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: E. G. Tourigny

Dated: July 13, 1989