December 20, 1988

Docket No. 50-325

DISTRIBUTION
See attached page

Mr. E. E. Utley
Senior Executive Vice President
Power Supply and Engineering & Construction
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

SUBJECT: ISSUANCE OF AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1, REGARDING REVISED INSTRUMENT TAG NUMBERS (TAC NO. 68459)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 121 to Facility Operating License No. DPR-71 for the Brunswick Steam Electric Plant, Unit 1. The amendment consists of changes to the Technical Specifications in response to your submittal dated June 9, 1988.

The amendment contains changes to Item 2, "Reactor Vessel Water Level," in Technical Specifications (TS) Tables 3.3.5.2-1 and 4.3.5.2-1 revising instrument tag numbers B21-LI-R604AX and B21-LT-N026A to B21-LI-R604BX and B21-LT-N026B, respectively.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

orignial signed by E. Tourigny

Edmond G. Tourigny, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 121 to License No. DPR-71
2. Safety Evaluation

cc w/enclosures:
See next page

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OFC	:LA:PD21:DRPR	:PM:PD21:DRPR	:PE:PD21:DRPR	:PM:PD21:DRPR	:D:PD21:DRPR	:	:
NAME	: PAnderson	:Etourigny:ch	:BMozafari	:BBuckley	:EAdensam	:	:
DATE	: 11/16/88	: 11/ 3/88	: 11/16/88	: 11/16/88	: 11/17/88	:	:

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cc:

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AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK, UNIT 1

Docket File
NRC PDR
Local PDR
PDII-1 Reading
S. Varga (14E4)
G. Lainas
E. Adensam
P. Anderson
E. Tourigny
B. Mozafari
B. Buckley
OGC
D. Hagan (MNBB 3302)
E. Jordan (MNBB 3302)
B. Grimes (9A2)
T. Barnhart (4) (P1-137)
W. Jones (P-130A)
E. Butcher (11F23)
J. Craig (8D1)
ACRS (10)
GPA/PA
ARM/LFMB

cc: Licensee/Applicant Service List

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated June 9, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 121, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

original signed by E. Adensam

Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 20, 1988

OFC	:LA:PD21:DRPR	:PM:PD21:DRPR	:PE:PD21:DRPR	:PM:PD21:DRPR	:SPLB:DEST	:OGC	:D:PD21:DRPR
NAME	:PAnderson	:ECourigny:	:BMozafari	:BBuckley	:JCraig	:	:EAdensam
DATE	:11/16/88	:11/3/88	:11/16/88	:11/16/88	:11/17/88	:11/30/88	:12/20/88

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ATTACHMENT TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages	Insert Pages
3/4 3-48	3/4 3-48
3/4 3-49	3/4 3-49

TABLE 3.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

FUNCTIONAL UNIT AND INSTRUMENT NUMBER	READOUT LOCATION	MINIMUM CHANNELS OPERABLE
1. Reactor Vessel Pressure (C32-PI-3332 and C32-PT-3332)	RSP*	1
2. Reactor Vessel Water Level (B21-LT-N017D-3, B21-LSH-N017D-3) (B21-LI-3331, B21-LI-R604BX, B21-LT-3331, B21-LT-N026B)	RSP*	1
3. Suppression Chamber Water Level (CAC-LI-3342 and CAC-LT-3342)	RSP*	1
4. Suppression Chamber Water Temperature (CAC-TR-778-7)	RSP*	1
5. Drywell Pressure (CAC-PI-3341 and CAC-PT-3341)	RSP*	1
6. Drywell Temperature (CAC-TR-778-1,3,4)	RSP*	1
7. Residual Heat Removal Head Spray Flow (E11-FT-3339 and E11-FI-3339)	RSP*	1
8. Residual Heat Removal System Flow (E11-FT-3338, E11-FI-3338, and E11-FY-3338)	RSP*	1
9. Residual Heat Removal Service Water Discharge Differential Pressure (E11-PDT-N002BX and E11-PDI-3344)	RSP*	1

*Remote Shutdown Panel, Reactor Building 20' Elevation

TABLE 4.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT AND INSTRUMENT NUMBER	CHANNEL CHECK	CHANNEL CALIBRATION
1. Reactor Vessel Pressure (C32-PI-3332 and C32-PT-3332)	M	Q
2. Reactor Vessel Water Level		
(B21-LT-NO17D-3, and B21-LSH-NO17D-3)	NA	Q
(B21-LI-3331, B21-LI-R604BX, B21-LT-3331, B21-LT-NO26B)	M	Q
3. Suppression Chamber Water Level (CAC-LI-3342 and CAC-LT-3342)	M	R
4. Suppression Chamber Water Temperature (CAC-TR-778-7)	M	R
5. Drywell Pressure (CAC-PI-3341 and CAC-PT-3341)	M	Q
6. Drywell Temperature (CAC-TR-778-1,3,4)	M	R
7. Residual Heat Removal Head Spray Flow (E11-FT-3339 and E11-FI-3339)	M	Q
8. Residual Heat Removal System Flow (E11-FT-3338, E11-FI-3338, and E11-FY-3338)	M	Q
9. Residual Heat Removal Service Water Discharge Differential Pressure (E11-PDT-NOO2BX and E11-PDI-3344)	M	Q

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-71

CAROLINA POWER & LIGHT COMPANY, et al.

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

DOCKET NO. 50-325

1.0 INTRODUCTION

By letter dated June 9, 1988, the Carolina Power & Light Company (CP&L) submitted a request for changes to the Brunswick Steam Electric Plant, Unit 1, Technical Specification (TS) Tables 3.3.5.2-1 and 4.3.5.2-1. The proposed amendment would change certain instrument numbers listed in the tables under Item 2, "Reactor Vessel Water Level." Level indicator number B21-LI-R604BX would replace B21-LI-R604AX and level transmitter number B21-LT-N026B would replace B21-LT-N026A.

These TS changes and plant modifications are needed to comply with 10 CFR Part 50, Appendix R, Section III.G, alternative shutdown capability requirements.

2.0 EVALUATION

The remote shutdown monitoring instrumentation provides sufficient instrumentation on the remote shutdown panel to monitor the status of the reactor and primary containment as well as operation of the reactor core isolation cooling (RCIC) and residual heat removal (RHR) systems. The remote shutdown panel is located in the Reactor Building. The information provided on the panel is either independent of the main control room instrumentation or is provided with isolation features so that malfunctions or fires in or near the control building will not affect its operation.

Currently, level transmitter loop B21-LT-N026A feeds reactor vessel water level indicator B21-LI-R604AX on the remote shutdown panel (directly), and indicated on B21-LI-R604A on the control panel (via the shutdown panel). Level transmitter loop B21-LT-N026B feeds only indicator B21-LI-R604B on the control panel.

The proposed change would have level transmitter B21-LT-N026B feeding both the control room indicator B21-LI-R604B and the remote shutdown panel indicator B21-LI-R604BX. The level signal would come to the control room via the hot shutdown panel. Level transmitter B21-LT-N026A would feed only indicator B21-LI-R604A in the control room.

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These modifications are being made to address alternate shutdown capability requirements associated with 10 CFR Part 50, Appendix R, Section III.G.

Currently, a Train A instrument, B21-LT-N026A, feeds the remote shutdown panel and a Train B instrument, B21-LT-N026B, feeds the control room. To satisfy Appendix R requirements, the modification is being made to make the Train A instrument feed the control room and the Train B instrument feed the remote shutdown panel. The cables are being run through the existing cable runs for Trains A and B. Appropriate barriers and equipment for the modified cable routes are being provided, as well.

Technical Specification Tables 3.3.5.2-1 and 4.3.5.2-1 currently list level indicator B21-LI-R604AX and level transmitter B21-LT-N026A under Item 2, "Reactor Vessel Water Level." The proposed change would replace the reference to indicator B21-LI-R604AX with a reference to indicator B21-LI-R604BX, and the reference to level transmitter B21-LT-N026A with a reference to transmitter B21-LT-R604B. Thus, level transmitter B21-LT-N026A would provide a vessel level signal to indicator B21-LI-R604A in the control room. Level transmitter B21-LT-N026B would provide a vessel level signal to indicator B21-LI-R604BX on the remote shutdown panel and, via the remote shutdown panel, indicator B21-LI-R604B in the control room.

The staff has reviewed the above analysis and agrees as to the appropriateness of the plant modification to meet alternate shutdown capability requirements. The staff also agrees that the Technical Specifications need to be changed to reflect the plant modifications, and that level indicator number B21-LI-R604BX would replace B21-LI-R604AX and level transmitter number B21-LT-N026B would replace B21-LT-N026A in Tables 3.3.5.2-1 and 4.3.5.2-1.

Based upon the above evaluation, the staff has determined that the amendment request is acceptable and complies with commitments made by the licensee with respect to Appendix R of 10 CFR Part 50.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20 and a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released off site, and that there should be no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration, and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The Commission made a proposed determination that this amendment involves no significant hazards consideration which was published in the Federal Register (53 FR 46138) on November 16, 1988, and consulted with the State of North Carolina. No public comments or requests for hearing were received, and the State of North Carolina did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: E. Tourigny, Project Directorate II-1
B. Mozafari, Project Directorate II-1

Dated: December 20, 1988