

March 6, 1984

Docket Nos. 50-325/324

Mr. E. E. Utley
Executive Vice President
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment Nos. 66 and 92 to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your submittal of September 7, 1982.

The amendments modify the Technical Specifications to provide a clarification of certain requirements, provide editorial and/or administrative corrections of certain requirements, and provide changes to reflect consistency with the actual plant design. The revisions are described as follows: revise the monthly channel check in the surveillance requirements, for Reactor Vessel Water Level to "Not Applicable" for instruments B21-LT-N017D-3 and B21-LSH-N017D-3, revise the valve group number from 7 to 8 for the reactor vessel head spray isolation valves, and revise the valve group number from 8 to 2 for the RHR discharge isolation valves to radwaste and the RHR process sampling valves, revise the minimum number of flame, heat, and smoke instruments required to be operable in their defined fire zones and add additional fire zones that have been established, revise the surveillance requirement for demonstrating Safety/Relief Valve (S/RV) operability, and revise the snubber list to reflect typographical corrections, snubber additions, and snubber deletions.

This amendment request also requested changes to the Technical Specifications regarding surveillance requirements for primary containment integrity and regarding by-passing the actuation of isolation functions associated with a loss of vacuum in the main turbine condenser. This portion of the amendment request will be addressed in a separate action.

B404060055 B40306
PDR ADDCK 05000324
P PDR

March 6, 1984

Mr. E. E. Utley

- 2 -

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Original Signed by /

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 66 to
License No. DPR-71
2. Amendment No. 92 to
License No. DPR-62
3. Safety Evaluation

cc w/enclosures:
See next page

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Mr. E. E. Utley
Carolina Power & Light Company
Brunswick Steam Electric Plant, Units 1 and 2

cc:

Richard E. Jones, Esquire
Carolina Power & Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

George F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Mr. Charles R. Dietz
Plant Manager
Post Office Box 458
Southport, North Carolina 28461

Mr. Franky Thomas, Chairman
Board of Commissioners
Post Office Box 249
Bolivia, North Carolina 28422

Mrs. Chrys Baggett
State Clearinghouse
Budget and Management
116 West Jones Street
Raleigh, North Carolina 27603

U. S. Environmental Protection
Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. W.
Atlanta, Georgia 30308

Resident Inspector
U. S. Nuclear Regulatory Commission
Star Route 1
Post Office Box 208
Southport, North Carolina 28461

James P. O'Reilly
Regional Administrator
Region II Office
U. S. Nuclear Regulatory Commission
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Dayne H. Browns, Chief
Radiation Protection Branch
Division of Facility Services
Department of Human Resources
Post Office Box 12200
Raleigh, North Carolina 27605



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated September 7, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

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PDR ADDCK 05000324
P PDR

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 66, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: March 6, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 66

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Revise the Appendix A Technical Specifications as follows:

<u>Remove</u>	<u>Insert</u>
3/4 3-48	3/4 3-48
3/4 3-49	3/4 3-49
3/4 3-60	3/4 3-60
3/4 3-61	3/4 3-61
3/4 4-4	3/4 4-4
3/4 6-17	3/4 6-17
3/4 7-14	3/4 7-14
3/4 7-16	3/4 7-16
3/4 7-18	3/4 7-18
3/4 7-19	3/4 7-19
3/4 7-20	3/4 7-20
3/4 7-23	3/4 7-23
3/4 7-25	3/4 7-25
3/4 7-26	3/4 7-26
3/4 7-27	3/4 7-27

TABLE 3.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>READOUT LOCATION</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Vessel Pressure (C32-PI-3332 and C32-PT-3332)	RSP*	1
2. Reactor Vessel Water Level (B21-LT-NO17D-3, B21-LSH-NO17D-3) (B21-LI-3331, B21-LI-R604AX, B21-LT-3331, B21-LT-NO26A)	RSP*	1
3. Suppression Chamber Water Level (CAC-LI-3342 and CAC-LT-3342)	RSP*	1
4. Suppression Chamber Water Temperature (CAC-TR-778-7)	RSP*	1
5. Drywell Pressure (CAC-PI-3341 and CAC-PT-3341)	RSP*	1
6. Drywell Temperature (CAC-TR-778-1,3,4)	RSP*	1
7. Residual Heat Removal Head Spray Flow (E11-FT-3339 and E11-FI-3339)	RSP*	1
8. Residual Heat Removal System Flow (E11-FT-3338, E11-FI-3338, and E11-FY-3338)	RSP*	1
9. Residual Heat Removal Service Water Discharge Differential Pressure (E11-PDT-NO02BX and E11-PDI-3344)	RSP*	1

*Remote Shutdown Panel, Reactor Building 20' Elevation

TABLE 4.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure (C32-PI-3332 and C32-PT-3332)	M	Q
2. Reactor Vessel Water Level (B21-LT-N017D-3, B21-LSH-N017D-3) (B21-LI-3331, B21-LI-R604AX, B21-LT-3331, B21-LT-N026A)	NA M	Q Q
3. Suppression Chamber Water Level (CAC-LI-3342 and CAC-LT-3342)	M	R
4. Suppression Chamber Water Temperature (CAC-TR-778-7)	M	R
5. Drywell Pressure (CAC-PI-3341 and CAC-PT-3341)	M	Q
6. Drywell Temperature (CAC-TR-778-1,3,4)	M	R
7. Residual Heat Removal Head Spray Flow (E11-FT-3339 and E11-FI-3339)	M	Q
8. Residual Heat Removal System Flow (E11-FT-3338, E11-FI-3338, and E11-FY-3338)	M	Q
9. Residual Heat Removal Service Water Discharge Differential Pressure (E11-PDT-N002BX and E11-PDI-3344)	M	Q

TABLE 3.3.5.7-1

FIRE DETECTION INSTRUMENTS

<u>INSTRUMENT LOCATION</u>		<u>MINIMUM INSTRUMENTS OPERABLE</u>		
		<u>FLAME</u>	<u>HEAT</u>	<u>SMOKE</u>
1. Reactor Building #1				
Zone 1	-17'	0	0	1
Zone 2	-17'	0	0	1
Zone 3	-17'	0	0	6
Zone 4	-17'	0	0	6
Zone 5	20'	0	0	12
Zone 6	20'	0	0	11
Zone 7	20'	0	0	10
Zone 8	50'	0	0	11
Zone 9	50'	0	0	15
Zone 10	80'	0	0	8
Zone 11	80'	0	0	10
Zone 12	98'	0	0	3
Zone 13	117'	0	0	1
Zone 14	117'	0	0	34
Zone 16	77'	0	0	4
2. Control Building				
Zone 1	70'	0	0	9
Zone 2	49'	0	0	4
Zone 3	49'	0	0	4
Zone 4	49'	0	0	13
Zone 5	49'	0	0	14
Zone 6	49'	0	0	6
Zone 7	23'	0	0	3
Zone 8	23'	0	0	3
Zone 9	23'	0	0	25
Zone 10	23'	0	0	24
Zone 11	23'	0	0	3
Zone 12	23'	0	0	3
Zone 13	49'	0	0	9
Zone 14	49'	0	0	9
Zone 15	70'	0	1	0
Zone 16	70'	0	1	0
3. Diesel Generator Building				
Zone 1	2'	0	0	25
Zone 2	2'	0	0	24
Zone 3	50'	0	0	9
Zone 4	23'	0	0	7
Zone 5	23'	0	0	5
Zone 6	23'	0	0	5

TABLE 3.3.5.7-1 (Continued)

<u>INSTRUMENT LOCATION</u>		<u>MINIMUM INSTRUMENTS OPERABLE</u>		
		<u>FLAME</u>	<u>HEAT</u>	<u>SMOKE</u>
3. Diesel Generator Building (Cont'd)				
Zone 7	23'	0	0	5
Zone 8	23'	0	0	5
Zone 9	23'	0	0	8
Zone 10	50'	0	0	9
4. Service Water Building				
Zone 1	4'	0	0	7
Zone 2	20'	0	0	6
5. AOG Building				
Zone 1	20'	0	0	2
Zone 2	20'	0	0	2
Zone 3	20'	1	5	1
Zone 4	37' - 49'	1	6	6

REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY/RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.4.2 The safety valve function of all reactor coolant system safety/relief valves shall be OPERABLE with lift settings within $\pm 1\%$ of the following values.*#

- 4 Safety-relief valves @ 1105 psig.
- 4 Safety-relief valves @ 1115 psig.
- 3 Safety-relief valves @ 1125 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With the safety valve function of one safety/relief valve inoperable, restore the inoperable safety valve function of the valve to OPERABLE status within 31 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With the safety valve function of two safety/relief valves inoperable, restore the inoperable safety valve function of at least one of the valves to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With the safety valve function of more than two safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.2 The safety valve function of each of the above required safety/relief valves shall be demonstrated OPERABLE in accordance with the Surveillance Requirements of Specification 4.0.5.

* The lift setting pressure shall correspond to ambient conditions of the valves at normal operating temperature and pressure.

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION</u>	<u>VALVE GROUP^{1/}</u>	<u>ISOLATION TIME (Seconds)</u>
Reactor vessel head spray isolation valves E11-F022 E11-F023	8	30
RHR shutdown cooling supply isolation valves E11-F008 E11-F009	8	30
RHR injection isolation valves E11-F015A, B	8	30
RHR discharge isolation valves to radwaste E11-F040 E11-F049	2	30
RHR process sampling valves E11-F079A, B E11-F080A, B	2	30

NOTE 1: See Specification 3.3.2, Table 3.3.2-1 for isolation signal that operates each valve group.

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Instrument Sensing System</u>				
1B21-701SS164	<u>Drywell</u> 104'	I	No	No
701SS167	104'	I	No	No
701SS169	100'	I	No	No
701SS170	103'	I	No	No
701SS171	99'	I	No	No
701SS172	101'	I	No	No
701SS175	100'	I	No	No
701SS177	94'	I	No	No
701SS178	97'	I	No	No
701SS179	96'	I	No	No
701SS184	88'	I	No	No
<u>Reactor Closed Cooling Water System</u>				
1RCC-32SS30	<u>Reactor Building</u> 55'	A	No	No
32SS45	60'	A	No	No
36SS78	54'	A	No	No
37SS79	54'	A	No	No
39SS80	59'	A	No	No
38SS81	54'	A	No	No
7SS112	57'	A	No	No
48SS167	59'	A	No	No
48SS168	58'	A	No	No
48SS169	60'	A	No	No
50SS272	4'	A	No	No
60SS121	<u>Drywell</u> 17'	I	No	No
60SS122	16'	I	No	No
65SS128	7'	I	No	No
65SS129	9'	I	No	No
71SS139	9'	I	No	No
73SS145	5'	I	No	No
19SS157	21'	I	No	No
19SS160	29'	I	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
	<u>Reactor Core Isolation Cooling System</u>			
1E51-4SS45	<u>Drywell</u>	31'	I	No
3SS46		39'	I	No
3SS47		39'	I	No
4SS66		39'	I	No
4SS68		40'	I	No
4SS69		40'	I	No
4SS70		39'	I	No
4SS71		36'	I	No
4SS72		31'	I	No
4SS73		30'	I	No
41SS51	<u>Reactor Building</u>	40'	A	No
42SS74		20'	A	No
42SS75		20'	A	No
42SS76		18'	A	No
42SS77		5'	A	No
42SS78		0'	A	No
42SS79		4'	A	No
42SS80		-13'	A	No
42SS81		-16'	A	No
42SS82		-9'	A	No
40SS83		-9'	A	No
40SS84		-9'	A	No
40SS85		-12'	A	No
40SS86		-9'	A	No
40SS87		-15'	A	No
40SS88		-13'	A	No
41SS89		41'	A	No
41SS95		-41'	A	No
19SS113		-17'	A	No
19SS114		-16'	A	No
49SS129		0'	A	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Nuclear Steam Vent System</u>				
1B21-44SS129	<u>Drywell</u> 104'	I	No	No
44SS131	93'	I	No	No
44SS134	99'	I	No	No
44SS136	97'	I	No	No
44SS137	96'	I	No	No
44SS138	95'	I	No	No
44SS141	87'	I	No	No
44SS142	87'	I	No	No
44SS143	87'	I	No	No
44SS146	87'	I	No	No
46SS147	82'	I	No	No
44SS149	85'	I	No	No
44SS150	83'	I	No	No
47SS155	75'	I	No	No
47SS156	78'	I	No	No
47SS157	75'	I	No	No
<u>Standby Liquid Control System</u>				
1C41-9SS4	<u>Drywell</u> 63'	I	No	No
9SS5	47'	I	No	No
9SS8	42'	I	No	No
9SS10	38'	I	No	No
9SS11	39'	I	No	No
9SS12	69'	I	No	No
9SS13	52'	I	No	No
9SS26	<u>Reactor Building</u> 72'	A	No	No
9SS27	72'	A	No	No
6SS34	84'	A	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Fuel Pool Cooling System</u>				
1G41-1SS22	<u>Reactor Building</u> 12'	A	No	No
1SS24	38'	A	No	No
1SS30	38'	A	No	No
12SS32	9'	A	No	No
12SS33	9'	A	No	No
15SS37	111'	A	No	No
20SS76	108'	A	No	No
19SS79	89'	A	No	No
22SS85	108'	A	No	No
12SS98	88'	A	No	No
6SS111	88'	A	No	No
7SS121	87'	A	No	No
5SS152	82'	A	No	No
<u>Reactor Recirculation System</u>				
1B32-SSA1	<u>Drywell</u> 8'	I	No	No
SSB1	81'	I	No	No
SSA2	11'	I	No	No
SSB2	11'	I	No	No
SSA3	11'	I	No	No
SSB3	11'	I	No	No
SSA4	21'	I	No	No
SSB4	21'	I	No	No
SSA5	21'	I	No	No
SSB5	21'	I	No	No
SSA6	27'	I	No	No
SSB6	27'	I	No	No
SSB9A	30'	I	No	No
SSB9B	30'	I	No	No
SSA10	24'	I	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Reactor Recirculation System (Continued)</u>				
SSA11	<u>Drywell (Cont'd)</u> 11'	I	No	No
SSB11	11'	I	No	No
SSA12A	30'	I	No	No
SSA12B	30'	I	No	No
SSB12A	30'	I	No	No
SSB12B	30'	I	No	No
<u>Reactor Vessel Instrumentation</u>				
1PS-3554	<u>Drywell</u> 32'	I	No	No
3558	32'	I	No	No
3561	32'	I	No	No
3562	60'	I	No	No
3567	63'	I	No	No
3570	32'	I	No	No
3613	32'	I	No	No
3617A	32'	I	No	No
3617B	32'	I	No	No
3751	34'	I	No	No
3752	34'	I	No	No
<u>Reactor Feedwater System</u>				
1B21-2SS3	<u>Drywell</u> 38'	I	No	No
2SS4	56'	I	No	No
3SS6	41'	I	No	No
3SS9	39'	I	No	No
3SS11	41'	I	No	No
3SS12	40'	I	No	No
3SS13	61'	I	No	No
5SS17	38'	I	No	No
5SS18	56'	I	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Reactor Feedwater System (Continued)</u>				
6SS20	<u>Drywell (Cont'd)</u> 41'	I	No	No
6SS23	39'	I	No	No
6SS25	41'	I	No	No
6SS26	40'	I	No	No
6SS27	63'	I	No	No
1SS227	34'	I	No	No
1SS228	38'	I	No	No
2SS229	53'	I	No	No
2SS230	62'	I	No	No
3SS231	40'	I	No	No
3SS232	36'	I	No	No
3SS233	40'	I	No	No
3SS234	48'	I	No	No
3SS235	63'	I	No	No
4SS236	34'	I	No	No
4SS237	38'	I	No	No
5SS238	53'	I	No	No
5SS239	61'	I	No	No
6SS240	41'	I	No	No
6SS241	36'	I	No	No
6SS242	39'	I	No	No
6SS243	48'	I	No	No
6SS244	61'	I	No	No
<u>Residual Heat Removal System</u>				
1E11-90SS267	<u>Drywell</u> 79'	I	No	No
90SS268	86'	I	No	No
90SS271	86'	I	No	No
90SS274	93'	I	No	No
90SS275	93'	I	No	No
90SS277	96'	I	No	No
90SS278	96'	I	No	No
90SS280	101'	I	No	No
90SS281	93'	I	No	No
90SS282	101'	I	No	No

BRUNSWICK - UNIT 1

3/4 7-20

Amendment No. 66

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Residual Heat Removal System (Continued)</u>				
1E11-132SS264	<u>Reactor Bldg.</u> 31'	A	No	No
21SS296	(Cont'd) 39'	A	No	No
21SS297	39'	A	No	No
47SS323	42'	A	No	No
47SS326	42'	A	No	No
47SS328	42'	A	No	No
49SS330	42'	A	No	No
49SS331	42'	A	No	No
49SS333	42'	A	No	No
49SS334	43'	A	No	No
49SS336	40'	A	No	No
128SS355	42'	A	No	No
49SS359	42'	A	No	No
127SS376	59'	A	No	No
128SS387	43'	A	No	No
2SS396	5'	A	No	No
2SS397	3'	A	No	No
5SS398	-41'	A	No	No
2SS399	-3'	A	No	No
5SS400	-3'	A	No	No
4SS401	-12'	A	No	No
5SS402	-12'	A	No	No
3SS403	-11'	A	No	No
6SS404	-12'	A	No	No
8SS405	-14'	A	No	No
6SS406	-14'	A	No	No
8SS407	-15'	A	No	No
12SS408	-14'	A	No	No
116SS409	-9'	A	No	No
113SS410	-9'	A	No	No
9SS411	-14'	A	No	No
109SS412	-14'	A	No	No
2SS413	0'	A	No	No
132SS414	43'	A	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Residual Heat Removal System (Continued)</u>				
1E11-89SS460	<u>Reactor Bldg.</u> 10'	A	No	No
89SS461	(Cont'd) 6'	A	No	No
53SS462	15'	A	No	No
53SS463	14'	A	No	No
53SS464	14'	A	No	No
53SS465	14'	A	No	No
53SS466	14'	A	No	No
50SS467	14'	A	No	No
50SS468	17'	A	No	No
18SS469	53'	A	No	No
18SS470	43'	A	No	No
89SS480	67'	A	No	No
89SS487	67'	A	No	No
89SS489	67'	A	No	No
89SS491	67'	A	No	No
91SS499	69'	A	No	No
91SS500	57'	A	No	No
56SS504	14'	A	No	No
56SS505	7'	A	No	No
56SS506	3'	A	No	No
56SS507	3'	A	No	No
56SS508	4'	A	No	No
46SS509	8'	A	No	No
46SS510	11'	A	No	No
46SS511	10'	A	No	No
46SS512	-1'	A	No	No
58SS514	14'	A	No	No
49SS515	37'	A	No	No
49SS516	37'	A	No	No
56SS517	-5'	A	No	No
51SS546	32'	A	No	No
51SS547	28'	A	No	No
115SS549	31'	A	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Residual Heat Removal System (Continued)</u>				
1E11-54SS551	<u>Reactor Bldg.</u> 31'	A	No	No
54SS552	(Cont'd) 28'	A	No	No
98SS554	32'	A	No	No
58SS563	7'	A	No	No
58SS565	13'	A	No	No
58SS566	6'	A	No	No
107SS573	15'	A	No	No
69SS574	6'	A	No	No
91SS575	53'	A	No	No
68SS577	8'	A	No	No
53SS596	14'	A	No	No
50SS597	26'	A	No	No
<u>Service Water System</u>				
1SW-133SS22	<u>Reactor Building</u> -6'	A	No	No
110SS35	-5'	A	No	No
174SS70	42'	A	No	No
173SS72	14'	A	No	No
142SS74	40'	A	No	No
142SS75	40'	A	No	No
142SS82	70'	A	No	No
140SS86	45'	A	No	No
153SS102	44'	A	No	No
173SS110	48'	A	No	No
173SS114	70'	A	No	No
103SS117	41'	A	No	No
103SS121	38'	A	No	No
103SS126	60'	A	No	No
103SS127	57'	A	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Service Water System (Continued)</u>				
1SW-106SS131	Reactor Bldg.	60'	A	No
100SS145	(Cont'd)	59'	A	No
100SS149		60'	A	No
106SS151		59'	A	No
106SS156		60'	A	No
142SS163		60'	A	No
142SS164		8'	A	No
142SS165		8'	A	No
175SS166A		42'	A	No
175SS166B		42'	A	No
140SS167		42'	A	No
142SS168		58'	A	No
142SS169		71'	A	No
174SS174		42'	A	No
173SS175		30'	A	No
133SS177		-5'	A	No
100SS193		62'	A	No
106SS211		60'	A	No
106SS212		60'	A	No
106SS213		59'	A	No
106SS214		60'	A	No
127SS215		17'	A	No
123SS216		17'	A	No



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated September 7, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 92, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: March 6, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 92

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Revise the Appendix A Technical Specifications as follows:

<u>Remove</u>	<u>Insert</u>
3/4 3-48	3/4 3-48
3/4 3-49	3/4 3-49
3/4 3-60	3/4 3-60
3/4 3-61	3/4 3-61
3/4 4-4	3/4 4-4
3/4 6-17	3/4 6-17
3/4 7-11	3/4 7-11
3/4 7-12	3/4 7-12
3/4 7-19	3/4 7-19
3/4 7-22	3/4 7-22
3/4 7-28	3/4 7-28
3/4 7-30	3/4 7-30
3/4 7-31	3/4 7-31
3/4 7-33	3/4 7-33

TABLE 3.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>READOUT LOCATION</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Vessel Pressure (C32-PI-3332 and C32-PT-3332)	RSP*	1
2. Reactor Vessel Water Level (B21-LT-N017D-3, B21-LSH-N017D-3) (B21-LI-3331, B21-LI-R604AX, B21-LT-3331, B21-LT-N026A)	RSP*	1
3. Suppression Chamber Water Level (CAC-LI-3342 and CAC-LT-3342)	RSP*	1
4. Suppression Chamber Water Temperature (CAC-TR-778-7)	RSP*	1
5. Drywell Pressure (CAC-PI-3341 and CAC-PT-3341)	RSP*	1
6. Drywell Temperature (CAC-TR-778-1,3,4)	RSP*	1
7. Drywell Oxygen Concentration (CAC-AT-1259-2)	Local Panel	1
8. Residual Heat Removal Head Spray Flow (E11-FT-3339 and E11-FI-3339)	RSP*	1
9. Residual Heat Removal System Flow (E11-FT-3338, E11-FI-3338, and E11-FY-3338)	RSP*	1
10. Residual Heat Removal Service Water Discharge Differential Pressure (E11-PDT-N002BX and E11-PDI-3344)	RSP*	1

*Remote Shutdown Panel, Reactor Building 20' Elevation

TABLE 4.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure (C32-PI-3332 and C32-PT-3332)	M	Q
2. Reactor Vessel Water Level (B21-LT-N017D-3, B21-LSH-N017D-3) (B21-LI-3331, B21-LI-R604AX, B21-LT-3331, B21-LT-N026A)	NA M	Q Q
3. Suppression Chamber Water Level (CAC-LI-3342 and CAC-LT-3342)	M	R
4. Suppression Chamber Water Temperature (CAC-TR-778-7)	M	R
5. Drywell Pressure (CAC-PI-3341 and CAC-PT-3341)	M	Q
6. Drywell Temperature (CAC-TR-778-1,3,4)	M	R
7. Drywell Oxygen Concentration (CAC-AT-1259-2)	M	Q
8. Residual Heat Removal Head Spray Flow (E11-FT-3339 and E11-FI-3339)	M	Q
9. Residual Heat Removal System Flow (E11-FT-3338, E11-FI-3338, and E11-FY-3338)	M	Q
10. Residual Heat Removal Service Water Discharge Differential Pressure (E11-PDT-N002BX and E11-PDI-3344)	M	Q

BRUNSWICK - UNIT 2

3/4 3-49

Amendment No. 92

TABLE 3.3.5.7-1

FIRE DETECTION INSTRUMENTS

<u>INSTRUMENT LOCATION</u>		<u>MINIMUM INSTRUMENTS OPERABLE</u>		
		<u>FLAME</u>	<u>HEAT</u>	<u>SMOKE</u>
1. Reactor Building #2				
Zone 1	-17'	0	0	1
Zone 2	-17'	0	0	1
Zone 3	-17'	0	0	6
Zone 4	-17'	0	0	6
Zone 5	20'	0	0	12
Zone 6	20'	0	0	10
Zone 7	20'	0	0	9
Zone 8	50'	0	0	11
Zone 9	50'	0	0	15
Zone 10	80'	0	0	9
Zone 11	80'	0	0	10
Zone 12	98'	0	0	3
Zone 13	117'	0	0	1
Zone 14	117'	0	0	34
Zone 16	77'	0	0	4
2. Control Building				
Zone 1	70'	0	0	9
Zone 2	49'	0	0	4
Zone 3	49'	0	0	4
Zone 4	49'	0	0	13
Zone 5	49'	0	0	14
Zone 6	49'	0	0	6
Zone 7	23'	0	0	3
Zone 8	23'	0	0	3
Zone 9	23'	0	0	25
Zone 10	23'	0	0	24
Zone 11	23'	0	0	3
Zone 12	23'	0	0	3
Zone 13	49'	0	0	9
Zone 14	49'	0	0	9
Zone 15	70'	0	1	0
Zone 16	70'	0	1	0
3. Diesel Generator Building				
Zone 1	2'	0	0	25
Zone 2	2'	0	0	24
Zone 3	50'	0	0	9
Zone 4	23'	0	0	7
Zone 5	23'	0	0	5
Zone 6	23'	0	0	5

TABLE 3.3.5.7-1 (Continued)

<u>INSTRUMENT LOCATION</u>		<u>MINIMUM INSTRUMENTS OPERABLE</u>		
		<u>FLAME</u>	<u>HEAT</u>	<u>SMOKE</u>
3. Diesel Generator Building (Cont'd)				
Zone 7	23'	0	0	5
Zone 8	23'	0	0	5
Zone 9	23'	0	0	8
Zone 10	50'	0	0	9
4. Service Water Building				
Zone 1	4'	0	0	7
Zone 2	20'	0	0	6
5. AOG Building				
Zone 1	20'	0	0	2
Zone 2	20'	0	0	2
Zone 3	20'	1	5	1
Zone 4	37' - 49'	1	6	6

REACTOR COOLANT SYSTEM3/4.4.2 SAFETY/RELIEF VALVESLIMITING CONDITION FOR OPERATION

3.4.2 The safety valve function of all reactor coolant system safety/relief valves shall be OPERABLE with lift settings within $\pm 1\%$ of the following values.*

- 4 Safety-relief valves @ 1105 psig.
- 4 Safety-relief valves @ 1115 psig.
- 3 Safety-relief valves @ 1125 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With the safety valve function of one safety/relief valve inoperable, restore the inoperable safety valve function of the valve to OPERABLE status within 31 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With the safety valve function of two safety/relief valves inoperable, restore the inoperable safety valve function of at least one of the valves to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With the safety valve function of more than two safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.2 The safety valve function of each of the above required safety/relief valves shall be demonstrated OPERABLE in accordance with the Surveillance Requirements of Specification 4.0.5.

* The lift setting pressure shall correspond to ambient conditions of the valves at normal operating temperature and pressure.

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION</u>	<u>VALVE GROUP^{1/}</u>	<u>ISOLATION TIME (Seconds)</u>
Reactor vessel head spray isolation valves E11-F022 E11-F023	8	30
RHR shutdown cooling supply isolation valves E11-F008 F11-F009	8	30
RHR injection isolation valves E11-F015A, B	8	30
RHR discharge isolation valves to radwaste E11-F040 E11-F049	2	30
RHR process sampling valves E11-F079A, B E11-F080A, B	2	30

NOTE 1: See Specification 3.3.2, Table 3.3.2-1 for isolation signal that operates each valve group.

TABLE 3.7.5-1

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
	<u>Core Spray System</u>			
2E21-2SS32	<u>Reactor Building</u>	66'	A	No
25SS91		-6'	A	No
2SS16		0'	A	No
2SS17		13'	A	No
15SS19		-3'	A	No
15SS20		-3'	A	No
28SS23		-4'	A	No
25SS96		-6'	A	No
40SS106		-12'	A	No
40SS107		-12'	A	No
39SS108		-12'	A	No
39SS109		-12'	A	No
2SS31		68'	A	No
6SS41		70'	A	No
6SS42		69'	A	No
2SS18		14'	A	No
2E21-3SS46	<u>Drywell</u>	63'	I	No
3SS47		63'	I	No
3SS48		65'	I	No
3SS49		66'	I	No
7SS53		63'	I	No
7SS54		63'	I	No
7SS55		65'	I	No
7SS56		66'	I	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Reactor Water Cleanup System</u>				
2G31-1SS3	<u>Drywell</u> 54'	I	No	No
<u>Condensate Drains System</u>				
2B21-51SS103	<u>Drywell</u> 29'	I	No	No
51SS105	26'	I	No	No
51SS106	18'	I	No	No
50SS109	31'	I	No	No
50SS111	28'	I	No	No
51SS113	23'	I	No	No
51SS115	24'	I	No	No
51SS118	24'	I	No	No
<u>High Pressure Coolant Injection System</u>				
2E41-4SS44	<u>Drywell</u> 40'	I	No	No
4SS45	35'	I	No	No
4SS47	40'	I	No	No
4SS49	37'	I	No	No
4SS50	40'	I	No	No
4SS51	30'	I	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Nuclear Steam Vent System</u>				
2B21-44SS129	<u>Drywell</u> 104'	I	No	No
44SS131	93'	I	No	No
44SS134	99'	I	No	No
44SS136	97'	I	No	No
44SS137	96'	I	No	No
44SS138	95'	I	No	No
44SS141	87'	I	No	No
44SS142	87'	I	No	No
44SS143	87'	I	No	No
44SS146	87'	I	No	No
46SS147	82'	I	No	No
44SS149	85'	I	No	No
44SS150	83'	I	No	No
47SS155	75'	I	No	No
47SS156	78'	I	No	No
47SS157	75'	I	No	No
<u>Standby Liquid Control System</u>				
2C41-9SS4	<u>Drywell</u> 63'	I	No	No
9SS5	47'	I	No	No
9SS8	42'	I	No	No
9SS10	38'	I	No	No
9SS11	39'	I	No	No
9SS12	69'	I	No	No
9SS13	52'	I	No	No
2C41-9SS26	<u>Reactor Building</u> 72'	A	No	No
9SS27	72'	A	No	No
6SS34	84'	A	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Reactor Feedwater System</u>				
2B21-2SS3	<u>Drywell</u> 38'	I	No	No
2SS4	56'	I	No	No
3SS6	41'	I	No	No
3SS9	39'	I	No	No
3SS11	41'	I	No	No
3SS12	40'	I	No	No
3SS13	61'	I	No	No
5SS17	38'	I	No	No
5SS18	56'	I	No	No
6SS20	41'	I	No	No
6SS23	39'	I	No	No
6SS25	41'	I	No	No
6SS26	40'	I	No	No
6SS27	63'	I	No	No
1SS227	34'	I	No	No
1SS228	38'	I	No	No
2SS229	53'	I	No	No
2SS230	62'	I	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Residual Heat Removal System (Continued)</u>				
2E11-60SS440	<u>Reactor Bldg. (Cont'd)</u>	13'	A	No
65SS441		3'	A	No
65SS442		3'	A	No
60SS443		11'	A	No
73SS444		2'	A	No
21SS445		5'	A	No
68SS448		13'	A	No
75SS449		2'	A	No
61SS450		7'	A	No
60SS451		13'	A	No
60SS452		13'	A	No
60SS453		10'	A	No
60SS454		10'	A	No
89SS459		11'	A	No
89SS460		10'	A	No
89SS461		6'	A	No
53SS462		15'	A	No
53SS463		14'	A	No
53SS464		14'	A	No
53SS465		14'	A	No
53SS466		14'	A	No
50SS467		14'	A	No
50SS468		17'	A	No
56SS504		14'	A	No
56SS505		7'	A	No
56SS506		3'	A	No
56SS507		3'	A	No
56SS508		4'	A	No
46SS509		8'	A	No
46SS510		11'	A	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Service Water System</u>				
2SW-133SS22	<u>Reactor Building</u>	-6'	A	No
110SS35		-5'	A	No
173SS72		14'	A	No
142SS164		8'	A	No
142SS165		8'	A	No
133SS176		14'	A	No
133SS177		-5'	A	No
142SS74		40'	A	No
142SS75		40'	A	No
140SS86		45'	A	No
153SS102		44'	A	No
153SS109		44'	A	No
173SS110		48'	A	No
153SS115		44'	A	No
103SS121		38'	A	No
140SS167		42'	A	No
173SS175		30'	A	No
142SS82		70'	A	No
173SS114		70'	A	No
103SS126		60'	A	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Service Water System (Continued)</u>				
2SW-103SS127	<u>Reactor Bldg. (Cont'd)</u> 57'	A	No	No
106SS131	60'	A	No	No
100SS145	59'	A	No	No
100SS149	60'	A	No	No
106SS151	59'	A	No	No
174SS70	42'	A	No	No
106SS156	60'	A	No	No
142SS168	58'	A	No	No
142SS169	71'	A	No	No
100SS193	62'	A	No	No
106SS211	60'	A	No	No
106SS212	60'	A	No	No
106SS213	59'	A	No	No
106SS214	60'	A	No	No
103SS117	41'	A	No	No
142SS163	0	A	No	No
175SS166A	42'	A	No	No
175SS166B	42'	A	No	No
109SS173	14'	A	No	No
174SS174	43'	A	No	No

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Steam Relief Discharge System (Continued)</u>				
2B21-33SS251	<u>Drywell (Cont'd)</u> 32'	I	No	No
20SS256	44'	I	No	No
20SS257	37'	I	No	No
20SS258	36'	I	No	No
20SS260	18'	I	No	No
20SS261	20'	I	No	No
20SS262	15'	I	No	No
27SS264	10'	I	No	No
27SS266	17'	I	No	No
27SS267	18'	I	No	No
27SS269	36'	I	No	No
27SS270	44'	I	No	No
27SS272	42'	I	No	No
58SS275	18'	I	No	No
58SS276	30'	I	No	No
58SS277	31'	I	No	No
58SS279	36'	I	No	No
58SS280	37'	I	No	No
58SS281	38'	I	No	No
12SS286	39'	I	No	No
12SS287	39'	I	No	No
12SS288	44'	I	No	No
12SS289	39'	I	No	No
12SS290	43'	I	No	No
12SS292	42'	I	No	No
34SS296	35'	I	No	No
34SS297	35'	I	No	No
34SS298	44'	I	No	No
34SS299	39'	I	No	No
34SS300	44'	I	No	No



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 66 TO FACILITY LICENSE NO. DPR-71 AND
AMENDMENT NO. 92 TO FACILITY LICENSE NO. DPR-62
CAROLINA POWER & LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
DOCKET NOS. 50-325 AND 50-324

INTRODUCTION

By letter dated September 7, 1982 Carolina Power & Light Company (CP&L) requested eight miscellaneous revisions to the Technical Specifications (TS) for the Brunswick Unit 1 and 2 operating licenses. The staff has completed the safety evaluation of six of the requested changes. These amendments would modify certain TS requirements, to provide a clarification of certain requirements, provide editorial and/or administrative corrections of certain requirements, and provide changes to reflect consistency with the actual plant design. The revisions are described as follows: revise the monthly channel check in the surveillance requirements, for Reactor Vessel Water Level to "Not Applicable" for instruments B21-LT-N017D-3 and B21-LSH-N017D-3, revise the valve group number from 7 to 8 for the reactor vessel head spray isolation valves, and revise the valve group number from 8 to 2 for the RHR discharge isolation valves to radwaste and the RHR process sampling valves, revise the minimum number of flame, heat, and smoke instruments required to be operable in their defined fire zones and add additional fire zones that have been established, revise the surveillance requirement for demonstrating Safety/Relief Valve (S/RV) operability, and revise the snubber list to reflect typographical corrections, snubber additions, and snubber deletions.

This amendment request also requested changes to the Technical Specifications regarding surveillance requirements for primary containment integrity and regarding by-passing the actuation of isolation functions associated with a loss of vacuum in the main turbine condenser. This portion of the amendment request will be addressed in a separate action.

EVALUATION

Remote Shutdown Monitoring Instrumentation (Units 1 and 2)

The Technical Specifications define CHANNEL CHECK as a qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from

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independent instrument channels measuring the same parameter. This requires that there be indication of the measured parameter. Table 3.3.5.2-1 (Remote Shutdown Monitoring Instrumentation) and Table 4.3.5.2-1 (Remote Shutdown Monitoring Instrumentation Surveillance Requirements) list reactor vessel water level transmitter B21-LT-N017D-3 and its associated level switch B21-LSH-N017D-3 and water level transmitters B21-LT-3331 and B21-LT-N026A and their associated level indicators B21-LI-3331 and B21-HI-R604AX.

The licensee has stated that the actual plant design is such that the reactor vessel water level transmitter B21-LT-N017D-3 and level switch B21-LSH-N017D3 input to bistable logic and do not provide input to an indicator. The bistable logic functions to cutoff reactor core isolation cooling (RCIC) on high reactor vessel water level. Since it is not possible to perform a interchannel comparison of display indications the request is to change the testing requirement to "Not Applicable" in Table 4.3.5.2-1. The requirements for a quarterly channel calibration would remain unchanged.

In addition, Tables 3.3.5.2-1 and 4.3.5.2-1 have been modified by segregating the reactor vessel level instrumentation into groups to reflect the channels that have indication and those that do not. This clarifies the Technical Specifications to reflect the plant design.

We find the request to modify Tables 3.3.5.2-1 and 4.3.5.2-1 to be acceptable based on the above discussion and since it is consistent with the surveillance requirements established by the Standard Technical Specifications for BWR plants.

Primary Containment Isolation Valves (Unit 2 only)

Table 3.6.3.1 of the Unit 2 TS indicates the residual heat removal (RHR) discharge isolation valves to radwaste (EII-F040 and EII-F049) and the RHR process sampling valves (EII-F079A, B and EII-F080A, B) are group 8 valves. In fact, these valves are group 2 valves. The revision from 8 to 2 for the valve groups for those valves, in accordance with the proposed TS page in Attachment 2 to CP&L's September 7, 1982 letter is acceptable since the revision provides consistency between TS information and the actual approved plant design.

Primary Containment Isolation Valves (Unit 1 only)

The TS revision for Unit 2 primary containment isolation valves is also applicable to the same valves for Unit 1 and is acceptable for Table 3.6.3.1 of the Unit 1 TS. An additional revision to Table 3.6.3.1 of the Unit 1 TS is required to correct a typographical error that occurred during the transition from custom TS to BWR Standard TS. Specifically, the revision from "7" to "8" has been reviewed by the staff for the valve groups for the reactor vessel head spray isolation valves (EII-F022 and

EII-F023) in accordance with the proposed TS page in Attachment 3 to CP&L's September 7, 1983 letter. The change is an administrative change that corrects the TS and is therefore acceptable.

Fire Detection Instrumentation (Units 1 and 2)

TS Table 3.3.5.7-1 lists the fire zones and the minimum number of operable flame, heat and smoke instruments for each zone. The number of installed fire detection instruments has been modified and the fire zones have been redefined by the licensee. Revisions to the applicable TS sections are necessary to reflect these changes. Accordingly, the proposed revisions to TS Table 3.3.5.7-1 for Units 1 and 2 in Attachment 4 to CP&L's September 7, 1982 letter have been reviewed by the staff and are acceptable since the revision provides consistency between the TS requirements and the actual plant design.

Safety/Relief Valves (S/RVs) (Units 1 and 2)

The three-stage S/RVs, which are equipped with bellows, have been replaced with two-stage S/RVs which are not equipped with bellows. TS 4.4.2 requires the demonstration of S/RV operability through a bellows integrity check once every 24 hours. This check is no longer applicable to Brunswick Units 1 and 2 and cannot be performed. Therefore, on this basis, we find acceptable CP&L's proposed change to TS 4.4.2 which would delete the bellows integrity check and add the provision that S/RV operability be demonstrated in accordance with the requirements of TS 4.0.5. The proposed page revisions for Units 1 and 2 TS in Attachment 5 to CP&L's September 7, 1982 letter have been reviewed by the staff and are acceptable.

Safety-Related Hydraulic Snubbers (Units 1 and 2)

TS Table 3.7.5-1 provides information on safety-related hydraulic snubbers. Subsequent to publication of the current Table 3.7.5-1, the licensee has performed plant modifications resulting in the rerouting or removal of some system lines. This in turn has caused the relocation or removal of selected safety-related hydraulic snubbers. The licensee has also discovered typographical errors in some snubber numbers accessibility information in the current Table 3.7.5-1. The staff has reviewed the proposed changes to Table 3.7.5-1 of the TS for Units 1 and 2 in Attachment 6 of CP&L's September 7, 1982 letter and finds them acceptable because they are administrative corrections and update the TS to be consistent with the actual plant design.

ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this

determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSIONS

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: L. L. Wheeler, M. Wigdor and D. Hoffman

Dated: March 6, 1984