

December 28, 1983

Mr. E. E. Utley
Executive Vice President
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. DPR-71 for the Brunswick Steam Electric Plant, Unit 1. The amendment consists of changes to the Technical Specifications in response to your application of June 16, 1982 as supplemented April 28 and August 10, 1983.

The amendment modifies the Technical Specifications to: (1) delete requirements for seismic snubbers on those portions of the control rod drive return line piping that have been removed, (2) add requirements regarding the operability, response time and surveillance of instrumentation installed as a result of certain NRC recommendations set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements," and (3) add requirements regarding the operability, response time and surveillance of instrumentation used for signalling a condition of high water level in the scram discharge piping.

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Marshall Grotenhuis, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

- 1. Amendment No. 63 to License No. DPR-71
- 2. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION:	Docket File	NRC PDR	LPDR	ORB#2 Reading	DEisenhut
SNorris	MGrotenhuis	OELD	SECY	LJHarmon	EJordan
JTaylor	TBarnhart-4	WJones	DBrinkman	ACRS-10	OPA-CMiles
RDiggs	NSIC	Gray			
*Please see previous concurrence page.					
DL:ORB#2	DL:ORB#2	DL:ORB#2	OELD	DL:AD:OR	
SNorris*	MGrotenhuis*	DVassallo*	<i>CB</i>	GLainas	
12/12/83	12/12/83	12/12/83	12/21/83	12/21/83	

12-19-83
CB

12/12/83

Mr. E. E. Utley
Carolina Power & Light Company
Brunswick Steam Electric Plant, Units 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated June 16, 1983, as supplemented April 28 and August 10, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

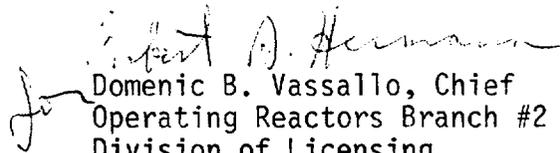
8401260255 831228
PDR ADOCK 05000325
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: December 28, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Remove

2-4
3/4 3-3
3/4 3-6a
3/4 3-8
3/4 3-48
3/4 3-49
3/4 3-50
3/4 3-51
3/4 3-51a
3/4 3-52
3/4 3-52a
3/4 6-30
3/4 7-12

Insert

2-4
3/4 3-3
3/4 3-6a
3/4 3-8
3/4 3-48
3/4 3-49
3/4 3-50
3/4 3-51
3/4 3-51a
3/4 3-52
3/4 3-52a
3/4 6-30
3/4 7-12

TABLE 2.2.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. Intermediate Range Monitor, Neutron Flux - High ⁽¹⁾ (C51-IRM-K601A,B,C,D,E,F,G,H)	\leq 120 divisions of full scale	\leq 120 divisions of full scale
2. Average Power Range Monitor (C51-APRM-CH.A,B,C,D,E,F)		
a. Neutron Flux - High, 15% ⁽²⁾	\leq 15% of RATED THERMAL POWER	\leq 15% of RATED THERMAL POWER
b. Flow-Biased Neutron Flux - High ⁽³⁾⁽⁴⁾	\leq (0.66 W + 54%)	\leq (0.66 W + 54%)
c. Fixed Neutron Flux - High ⁽⁴⁾	\leq 120% of RATED THERMAL POWER	\leq 120% of RATED THERMAL POWER
3. Reactor Vessel Steam Dome Pressure - High (B21-PTM-NO23A-1,B-1,C-1,D-1)	\leq 1045 psig	\leq 1045 psig
4. Reactor Vessel Water Level - Low, Level 1 (B21-LTM-NO17A-1,B-1,C-1,D-1)	\geq +162.5 inches*	\geq +162.5 inches*
5. Main Steam Line Isolation Valve - Closure ⁽⁵⁾ (B21-F022A,B,C,D; B21-F028A,B,C,D)	\leq 10% closed	\leq 10% closed
6. Main Steam Line Radiation - High (D12-RM-K603A,B,C,D)	\leq 3 x full power background	\leq 3.5 x full power background
7. Drywell Pressure - High (C71-PTM-NO02A-1,B-1,C-1,D-1)	\leq 2 psig	\leq 2 psig
8. Scram Discharge Volume Water Level - High (C11-LSH-NO13A,B,C,D) (C11-LSH-4516A,B,C,D)	\leq 109 gallons	\leq 109 gallons

TABLE 3.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM (a)</u>	<u>ACTION</u>
7. Drywell Pressure - High (C71-PT-N002A,B,C,D) (C71-PTM-N002A-1,B-1,C-1,D-1)	1, 2 ^(e)	2	6
8. Scram Discharge Volume Water Level - High (C11-LSH-N013A,B,C,D) (C11-LSH-4516A,B,C,D)	1, 2, 5 ^(f)	2	5
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	1(g)	4	8
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSL-1756,1757,1758,1759)	1(g)	2	8
11. Reactor Mode Switch in Shutdown Position (C71A-51)	1, 2, 3, 4, 5	1	9
12. Manual Scram (C71-A-S3A,B)	1, 2, 3, 4, 5	1	10

TABLE 3.3.1-2 (Continued)REACTOR PROTECTION SYSTEM RESPONSE TIMES

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>RESPONSE TIME (Seconds)</u>
7. Drywell Pressure - High (C71-PT-N002A,B,C,D) (C71-PTM-N002A-1,B-1,C-1,D-1)	NA
8. Scram Discharge Volume Water Level - High (C11-LSH-N013A,B,C,D) (C11-LSH-4516A,B,C,D)	NA
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	≤ 0.06
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSL-1756,1757,1758,1759)	≤ 0.08
11. Reactor Mode Switch in Shutdown Position (C71A-S1)	NA
12. Manual Scram (C71A-S3A,B)	NA

* Neutron detectors are exempt from response time testing. Response time shall be measured from detector output or from the input of the first electronic component in the channel.

TABLE 4.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION^(a)</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
5. Main Steam Line Isolation Valve - Closure (B21-F022A,B,C,D and B21-F028A,B,C,D)	NA	M	R ^(h)	1
6. Main Steam Line Radiation - High (D12-RM-K603A,B,C,D)	S	M ⁽¹⁾	R ^(j)	1, 2
7. Drywell Pressure - High (C71-PT-N002A,B,C,D) (C71-PTM-N002A-1,B-1,C-1,D-1)	NA ^(k)	NA	R ^(l)	1, 2
	D	M	M	1, 2
8. Scram Discharge Volume Water Level - High (C11-LSH-N013A,B,C,D) (C11-LSH-4516A,B,C,D)	NA	Q	R	1, 2, 5
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	NA	M	R ^(h)	1
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSI-1756,1757,1758,1759)	NA	M	R	1
11. Reactor Mode Switch in Shutdown Position (C71A-S1)	NA	R	NA	1, 2, 3, 4, 5
12. Manual Scram (C71A-S3A,B)	NA	Q	NA	1, 2, 3, 4, 5

TABLE 3.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>READOUT LOCATION</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Vessel Pressure (C32-PI-3332 and C32-PT-3332)	RSP*	1
2. Reactor Vessel Water Level (B21-LI-3331, B21-LI-R604AX, B21-LT-3331, B21-LT-N026A, B21-LT-N017D-3, and B21-LSH-N017D-3)	RSP*	1
3. Suppression Chamber Water Level (CAC-LI-3342 and CAC-LT-3342)	RSP*	1
4. Suppression Chamber Water Temperature (CAC-TR-778-7)	RSP*	1
5. Drywell Pressure (CAC-PI-3341 and CAC-PT-3341)	RSP*	1
6. Drywell Temperature (CAC-TR-778-1,3,4)	RSP*	1
7. Residual Heat Removal Head Spray Flow (E11-FT-3339 and E11-FI-3339)	RSP*	1
8. Residual Heat Removal System Flow (E11-FT-3338, E11-FI-3338, and E11-FY-3338)	RSP*	1
9. Residual Heat Removal Service Water Discharge Differential Pressure (E11-PDT-N002BX and E11-PDI-3344)	RSP*	1

*Remote Shutdown Panel, Reactor Building 20' Elevation

TABLE 4.3.5.2-1REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure (C32-PI-3332 and C32-PT-3332)	M	Q
2. Reactor Vessel Water Level (B21-LI-3331, B21-LI-R604AX, B21-LT-3331, B21-LT-NO26A, B21-NO17D-3, and B21-LSH-NO17D-3)	M	Q
3. Suppression Chamber Water Level (CAC-LI-3342 and CAC-LT-3342)	M	R
4. Suppression Chamber Water Temperature (CAC-TR-778-7)	M	R
5. Drywell Pressure (CAC-PI-3341 and CAC-PT-3341)	M	Q
6. Drywell Temperature (CAC-TR-778-1,3,4)	M	R
7. Residual Heat Removal Head Spray Flow (E11-FT-3339 and E11-FI-3339)	M	Q
8. Residual Heat Removal System Flow (E11-FT-3338, E11-FI-3338, and E11-FY-3338)	M	Q
9. Residual Heat Removal Service Water Discharge Differential Pressure (E11-PDT-NO02BX and E11-PDI-3344)	M	Q

INSTRUMENTATION

POST-ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.5.3 The post-accident monitoring instrumentation channels shown in Table 3.3.5.3-1 shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1 and 2.

ACTION:

- a. With the number of OPERABLE post-accident monitoring channels less than required by Table 3.3.5.3-1, either restore the inoperable channels to OPERABLE status within 31 days or be in at least HOT SHUTDOWN within the next 12 hours.
- b. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.5.3 Each of the above required post-accident monitoring instrumentation channels shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.5.3-1.

TABLE 3.3.5.3-1

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>MINIMUM NO. OF OPERABLE INSTRUMENT CHANNELS</u>
1. Reactor Vessel Pressure (B21-PI-RO04A,B; C32-LPR-R608; and C32-PT-NO05A,B)	2
2. Reactor Vessel Water Level (B21-LITS-NO26A,B; B21-LR-R615; B21-LI-R604A,B; B21-LT-NO37; and B21-LTM-NO37-1)	2
3. Suppression Chamber Water Level (CAC-LT-2601; CAC-LI-2601-1) (CAC-LT-2602; CAC-LR-2602)	2
4. Suppression Chamber Water Temperature (CAC-TR-1258-14, 21; and C91-P602)	2
5. Suppression Chamber Atmosphere Temperature (CAC-TR-1258-17 thru 20; and C91-P602)	2
6. Drywell Pressure (CAC-PI-4176; CAC-PT-4176; CAC-PR-1257-1; and CAC-PT-4175)	2
7. Drywell Temperature (CAC-TR-1258-1 thru 13, 22, 23, 24; and C91-P602)	2
8. Drywell Radiation (CAC-AR-1260; CAC-AQH-1260-1, 2, 3; CAC-AR-1261; CAC-AQH-1261-1, 2, 3; CAC-AR-1262; and CAC-AQH-1262-1, 2, 3)	2
9. Drywell Oxygen Concentration (CAC-AT-4409-37; CAC-AI-4409-40; CAC-X-XY-4348-2; CAC-X-XY-4349-2; CAC-AR-4409-41) (CAC-AT-4410-37; CAC-AI-4410-40; CAC-X-XY-4362-2; CAC-X-XY-4363-2; CAC-AR-4410-41)	2(a)

TABLE 3.3.5.3-1 (Continued)

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>MINIMUM NO. OF OPERABLE INSTRUMENT CHANNELS</u>
10. Drywell Hydrogen Concentration Analyzer and Monitor (CAC-AT-4409-38; CAC-AI-4409-32; CAC-X-XY-4348-1; CAC-X-XY-4349-1; CAC-AR-4409-42) (CAC-AT-4410-38; CAC-AI-4410-32; CAC-X-XY-4362-1; CAC-X-XY-4363-1; CAC-AR-4410-42)	2 ^(a)
11. Safety Relief Valve Position Indication: a. Primary - Sonic (B21-FY-4157 thru 4167) b. Secondary - Temp. (B21-TR-R614, points 1-11)	1/valve

(a) An OPERABLE instrument channel shall consist of the AT instrument and either the AI instrument or the XY-XY-AR instruments.

TABLE 4.3.5.3-1

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure (B21-PI-R004A,B; C32-LPR-R608; and C32-PT-N005A,B)	M	R
2. Reactor Vessel Water Level (B21-LITS-NO26A,B; B21-LR-R615; B21-LI-R604A,B; B21-LT-NO37; and B21-LTM-NO37-1)	M	R
3. Suppression Chamber Water Level (CAC-LT-2601; CAC-LI-2601-1; CAC-LT-2602; CAC-LR-2602)	M	R
4. Suppression Chamber Water Temperature (CAC-TR-1258-14, 21; and C91-P602)	M	R
5. Suppression Chamber Atmosphere Temperature (CAC-TR-1258-17 thru 20; and C91-P602)	M	R
6. Drywell Pressure (CAC-PI-4176; CAC-PT-4176; CAC-PR-1257-1; and CAC-PT-4175)	M	R
7. Drywell Temperature (CAC-TR-1258-1 thru 13, 22, 23, 24; and C91-P602)	M	R
8. Drywell Radiation (CAC-AR-1260; CAC-AQH-1260-1,2,3; CAC-AR-1261; CAC-AQH-1261-1,2,3; CAC-AR-1262; and CAC-AQH-1262-1,2,3)	M	R
9. Drywell Oxygen Concentration (CAC-AT-4409-37; CAC-AI-4409-40; CAC-X-XY-4348-2; CAC-X-XY-4349-2; CAC-AR-4409-41) (CAC-AT-4410-37; CAC-AI-4410-40; CAC-X-XY-4362-2; CAC-X-XY-4363-2; CAC-AR-4410-41)	M	R
10. Drywell Hydrogen Concentration Analyzer and Monitor (CAC-AT-4409-38; CAC-AI-4409-32; CAC-X-XY-4348-1; CAC-X-XY-4349-1; CAC-AR-4409-42) (CAC-AT-4410-38; CAC-AI-4410-32; CAC-X-XY-4362-1; CAC-X-XY-4363-1; CAC-AR-4410-42)	M	R

TABLE 4.3.5.3-1 (Continued)POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
11. Safety Relief Valve Position Indication		
a. Primary - Sonic (B21-FY-4157 thru 4167)	M	R
b. Secondary - Temp. (B21-TR-R614, points 1-11)	M	R

CONTAINMENT SYSTEMS

GAS ANALYZER SYSTEMS

LIMITING CONDITION FOR OPERATION

3.6.6.4 Two independent gas analyzer systems for the drywell and suppression chamber shall be OPERABLE with each system consisting of an oxygen analyzer and a hydrogen analyzer.

APPLICABILITY: OPERATIONAL CONDITION 1.

ACTION:

- a. With one oxygen and/or one hydrogen analyzer inoperable, restore at least two oxygen and two hydrogen analyzers to OPERABLE status within 31 days or be in at least STARTUP within the next 8 hours. The provisions of Specification 3.0.4 are not applicable.
- b. With no gas analyzer OPERABLE for oxygen and/or hydrogen, be in at least STARTUP within 8 hours.

SURVEILLANCE REQUIREMENTS

4.6.6.4 Each gas analyzer system (CAC-AT-4409, Division I and CAC-AT-4410, Division II) shall be demonstrated OPERABLE at least once per 92 days by performing a CHANNEL CALIBRATION using standard gas samples containing a nominal:

- a. Zero volume percent hydrogen, balance nitrogen.
- b. Seven to ten volume percent hydrogen, balance nitrogen.
- c. Twenty-five to thirty volume percent hydrogen, balance nitrogen.
- d. Zero volume percent oxygen, balance nitrogen.
- e. Seven to ten volume percent oxygen, balance nitrogen.
- f. Twenty to twenty-five volume percent oxygen, balance nitrogen.

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
<u>Condensate Drain System</u>				
1B21-51SS103	<u>Drywell</u> 29'	I	No	No
51SS105	29'	I	No	No
51SS106	26'	I	No	No
50SS109	18'	I	No	No
50SS111	31'	I	No	No
51SS113	28'	I	No	No
51SS115	23'	I	No	No
51SS118	24'	I	No	No
<u>Control Rod Drive System</u>				
1C11-16SS10	<u>Drywell</u> 72'	I	No	No
16SS11	72'	I	No	No
16SS12	72'	I	No	No
<u>High Pressure Coolant Injection System</u>				
1E41-4SS44	<u>Drywell</u> 40'	I	No	No
4SS45	35'	I	No	No
4SS47	40'	I	No	No
4SS49	37'	I	No	No
4SS50	50'	I	No	No
4SS51	30'	I	No	No
60SS9	<u>Reactor Building</u> 4'	A	No	No
6SS27	-5'	A	No	No
6SS28	1'	A	No	No
6SS30	-1'	A	No	No
6SS32	-5'	A	No	No
6SS33	1'	A	No	No
6SS35	-1'	A	No	No
6SS36	-5'	A	No	No



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 63 TO FACILITY LICENSE NO. DPR-71

CAROLINA POWER & LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

DOCKET NO. 50-325

1.0 Introduction

By letter dated June 16, 1982 as supplemented April 28, and August 10, 1983, the Carolina Power & Light Company (the licensee) submitted proposed changes to the Technical Specifications appended to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant (BSEP), Units 1 and 2.

The amendment would modify the Technical Specifications to: (1) delete requirements for seismic snubbers on those portions of the control rod drive return line piping that have been removed, (2) add requirements regarding the operability, response time and surveillance of instrumentation installed as a result of certain NRC recommendations set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements," and (3) add requirements regarding the operability, response time and surveillance of instrumentation used for signalling a condition of high water level in the scram discharge piping. Technical Specifications that would be changed to implement the recommendations in NUREG-0737 are associated with the following items as listed in NUREG-0737:

- II.B.3 Post accident sampling
- II.F.1 Accident monitoring
 - II.F.1.1 Noble gas monitor
 - II.F.1.2 Iodine/Particulate sampling
 - II.F.1.3 Containment high range monitor
 - II.F.1.5 Containment water level
 - II.F.1.6 Containment hydrogen

These NUREG-0737 items, as well as the removal of control rod drive return line piping and the addition of scram discharge volume instrumentation, are improvements that have been previously approved by the NRC. The changes to the Technical Specifications (TS) will be necessary administrative follow up actions essential to the implementation of these improvements.

2.0 Discussion

On August 17, 1982 Amendment No. 73 was issued for Unit 2 which revised the Technical Specification which deleted the snubbers from the control rod drive line and reflected the additional instrumentation to the scram discharge instruments. On September 22, 1983 Amendment No. 81 was issued for Unit 2 which reflected the instrumentation used to monitor the water level in the suppression chamber of the primary containment system (NUREG-0737 Item II.F.1.5).

On November 1, 1983 Generic Letter No. 83-36 was issued by the NRC. This letter includes guidance in the form of model Technical Specifications for, among other TMI requirements, II.B.3, II.F.1.1, II.F.1.5, and II.F.1.6. We have discussed this letter with the licensee staff. Items II.B.3, II.F.1.1, II.F.1.2, II.F.1.3, II.F.1.5, and II.F.1.6 will be resubmitted in response to GL 83-36. Items II.F.1.5 and II.F.1.6 for Unit 1 will be included in this amendment because they represent equipment as currently installed. In addition they will be resubmitted in response to GL 83-36.

3.0 Evaluation

3.1 Control Rod Drive (CRD) Return Line Snubbers

During the 1983 BSEP Unit 1 refueling outage, the licensee, in accordance with NUREG-0619 as forwarded by NRC generic letter dated November 13, 1980, partially cut and capped the CRD return line. The remainder of the modifications will be completed later and will be the subject of a separate action.

The licensee proposes to delete from the Technical Specifications those snubbers that supported the CRD return line prior to its removal. Since we are only concerned with snubbers that are required to insure the integrity of the reactor coolant system and safety related systems and since the snubbers in question no longer support a safety related system, we find the proposed Technical Specification change to be acceptable.

3.2 TMI Action Plan Requirements II.F.1.5 and II.F.1.6

Item 2 above delineates several TMI Action Plan Requirements that have been reviewed and approved but for which Technical Specifications have not been issued. Only II.F.1.5 and II.F.1.6 will be included in this amendment and only for Unit 1. II.F.1.5 has been completed for Unit 2 in Amendment No. 81 dated September 22, 1983 and II.F.1.6 will be completed at a later date for Unit 2.

NUREG-0737 Item II.F.1.5, concerns wide-range containment (suppression chamber) water level monitoring. The indication range for the Brunswick suppression pool was increased from (-6' to +6') to a new range of (-10' to +6') as a result of the requirements outlined in NUREG-0737.

The original configuration in which one transmitter drives both a narrow- and wide-range indicator was also changed. The original transmitter will continue to provide wide-range indication, and a new transmitter (CAC-LT-4177) was installed for narrow-range indication and annunciation. Only the wide-range indicator is required as part of the Post Accident Monitoring Instrumentation in the Technical Specifications. The proposed revisions to Tables 3.3.5.3-1 and 4.3.5.3-1 incorporate new instrument tag numbers due to the above mentioned modifications to the suppression chamber water level monitors.

NUREG-0737 Item II.F.1.6 concerns containment hydrogen monitoring. The proposed revisions to TS Tables 3.3.5.3-1 and 4.3.5.3-1 incorporate revised instrument numbers associated with the new drywell hydrogen and oxygen monitoring system. This modification involves the replacement of the existing drywell hydrogen/oxygen monitoring system with a new wide-range monitoring system; therefore, the revisions necessary to TS Tables 3.3.5.3-1 and 4.3.5.3-1 are the inclusion of new instrument numbers for item 9, drywell oxygen concentration, and item 10, drywell hydrogen concentration analyzer and monitor. The changes in tag numbers and replacement of old instruments reflect increased measurement ranges, improved design, remote calibration features, and improved moisture removal capabilities. Qualification pedigree ensures a safer, more accurate, and more reliable overall system performance.

The proposed change to TS Tables 3.3.5.3-1 and 4.3.5.3-1 also deletes item 7, drywell oxygen concentration, and renumbers items 8, 9, and 10. The deletion of item 7 from these TS tables is the result of plant modifications relating to NUREG-0737 Item II.F.1.6, containment hydrogen monitoring. The drywell oxygen concentration indication on the remote shutdown panel is being removed as part of the replacement of the existing drywell hydrogen/oxygen monitoring system with a new wide-range monitoring system. The proposed change is consistent with GE/BWR-4 Standard Technical Specification guidance which does not require drywell oxygen concentration indication as part of the remote shutdown monitoring instrumentation. The revision does not adversely impact plant safety since drywell oxygen concentration monitoring is available as part of the plant post-accident monitoring instrumentation listed in TS Tables 3.3.5.3-1 and 4.3.5.3-1.

We have reviewed the proposed Technical Specifications and find that they are administrative changes to reflect the installation of instrumentation as recommended in NUREG-0737. Therefore, we find the proposed Technical Specifications acceptable.

3.3 Scram Discharge Instrument Volume (SDIV) Diverse Instrumentation

During the 1983 BSEP Unit 1 refueling outage, the licensee installed additional, diverse instrumentation on the SDIVs. Installation of such instrumentation has been previously evaluated and approved as documented by the safety evaluation appended to an NRC generic letter dated December 9, 1980.

The licensee proposes to incorporate this additional instrumentation into the Technical Specifications. We have reviewed the proposed Technical Specification change and determined that the scram discharge volume (SDV) water level high operability, applicability, action and surveillance requirements remain unchanged, and that the proposed change only adds additional instrumentation. Therefore since the installation of SDIV diverse instrumentation has been previously approved and since the proposed change only adds additional SDV water level high instruments to the technical specifications, we find the proposed technical specification change to be acceptable.

4.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

5.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Dated: December 28, 1983