

February 10, 1984

Docket Nos. 50-325/324

Mr. E. E. Utley
Executive Vice President
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

On December 2, 1983 we issued Amendment No. 59 to Facility Operating License No. DPR-71 for the Brunswick Steam Electric Plant, Unit 1. This amendment inadvertently changed the trip set points on page 3/4 3-42 which had been established by Amendment No. 56 on June 28, 1983. On January 11, 1983 we issued Amendment Nos. 65 and 91 to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. These amendments contained an administrative error on page 3/4 6-2 for each unit. The pressure P for Type B and C test was inadvertently changed from the correct value, 49^apsig, to 25 psig.

The three pages have been corrected and are enclosed. We regret any inconvenience these errors may have caused you.

Sincerely,

Original signed by/
Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:
As stated

cc w/enclosures:
See next page

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PDR ADOCK 05000324
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Mr. E. E. Utley
Carolina Power & Light Company
Brunswick Steam Electric Plant, Units 1 and 2

cc:

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Dayne H. Browns, Chief
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Raleigh, North Carolina 27605

TABLE 3.3.4-1

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>APRM (C51-APRM-CH. A,B,C,D,E,F)</u>		
a. Upscale (Flow Biased)	$\leq (0.66W + 42\%) \frac{T^*}{MTPF}$	$\leq (0.66W + 42\%) \frac{T^*}{MTPF}$
b. Inoperative	NA	NA
c. Downscale	$> 3/125$ of full scale	$> 3/125$ of full scale
d. Upscale (Fixed)	$\leq 12\%$ of RATED THERMAL POWER	$\leq 12\%$ of RATED THERMAL POWER
2. <u>ROD BLOCK MONITOR (C51-RBM-CH.A,B)</u>		
a. Upscale	$\leq (0.66W + 41\%) \frac{T^*}{MTPF}$	$\leq (0.66W + 41\%) \frac{T^*}{MTPF}$
b. Inoperative	NA	NA
c. Downscale	$> 3/125$ of full scale	$> 3/125$ of full scale
3. <u>SOURCE RANGE MONITORS (C51-SRM-K600A,B,C,D)</u>		
a. Detector not full in	NA	NA
b. Upscale	$\leq 1 \times 10^5$ cps	$\leq 1 \times 10^5$ cps
c. Inoperative	NA	NA
d. Downscale	≥ 3 cps	≥ 3 cps
4. <u>INTERMEDIATE RANGE MONITORS (C51-IRM-K601A,B,C,D,E,F,G,H)</u>		
a. Detector not full in	NA	NA
b. Upscale	$\leq 108/125$ of full scale	$\leq 108/125$ of full scale
c. Inoperative	NA	NA
d. Downscale	$\geq 3/125$ of full scale	$\geq 3/125$ of full scale
5. <u>SCRAM DISCHARGE VOLUME (C11-LSH-NO13E)</u>		
a. Water Level - High	≤ 73 gallons	≤ 73 gallons

*T=2.43 for 8x8 fuel
T=2.39 for 8x8R fuel
T=2.39 for P8x8R fuel

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CONTAINMENT SYSTEMS

PRIMARY CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Primary containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of:
 1. Less than or equal to L_a , 0.5 percent by weight of the containment air per 24 hours at P_a , 49 psig, or
 2. Less than or equal to L_t , 0.357 percent by weight of the containment air per 24 hours at a reduced pressure of P_t , 25 psig.
- b. A combined leakage rate of less than or equal to $0.60 L_a$ for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, subject to Type B and C tests when pressurized to P_a , 49 psig.
- c. *Less than or equal to 11.5 scf per hour for any one main steam line isolation valve when tested at 25 psig.

APPLICABILITY: When PRIMARY CONTAINMENT INTEGRITY is required per Specification 3.6.1.1.

ACTION:

With:

- a. The measured overall integrated primary containment leakage rate exceeding $0.75 L_a$ or $0.75 L_t$, as applicable, or
- b. The measured combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, subject to Type B and C tests exceeding $0.60 L_a$, or
- c. The measured leakage rate exceeding 11.5 scf per hour for any one main steam line isolation valve,

restore:

- a. The overall integrated leakage rate(s) to less than or equal to $0.75 L_a$ or $0.75 L_t$, as applicable, and
- b. The combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, subject to Type B and C tests to less than or equal to $0.60 L_a$, and

* Exemption to Appendix "J" of 10 CFR 50.

CONTAINMENT SYSTEMS

PRIMARY CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Primary containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of:
 1. Less than or equal to L_a , 0.5 percent by weight of the containment air per 24 hours at P_a , 49 psig, or
 2. Less than or equal to L_t , 0.357 percent by weight of the containment air per 24 hours at a reduced pressure of P_t , 25 psig.
- b. A combined leakage rate of less than or equal to $0.60 L_a$ for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, subject to Type B and C tests when pressurized to P_a , 49 psig.
- c. *Less than or equal to 11.5 scf per hour for any one main steam line isolation valve when tested at 25 psig.

APPLICABILITY: When PRIMARY CONTAINMENT INTEGRITY is required per Specification 3.6.1.1.

ACTION:

With:

- a. The measured overall integrated primary containment leakage rate exceeding $0.75 L_a$ or $0.75 L_t$, as applicable, or
- b. The measured combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, subject to Type B and C tests exceeding $0.60 L_a$, or
- c. The measured leakage rate exceeding 11.5 scf per hour for any one main steam line isolation valve,

restore:

- a. The overall integrated leakage rate(s) to less than or equal to $0.75 L_a$ or $0.75 L_t$, as applicable, and
- b. The combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves*, subject to Type B and C tests to less than or equal to $0.60 L_a$, and

* Exemption to Appendix "J" of 10 CFR 50.