

Docket Nos. 50-325
and 50-324

May 6, 1981

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- HDenton
- JHeltemes, AEOD
- NSIC
- TERA (2)

Mr. J. A. Jones
 Senior Executive Vice President
 Carolina Power & Light Company
 336 Fayetteville Street
 Raleigh, North Carolina 27602



Dear Mr. Jones:

The Commission has issued the enclosed Amendment Nos. 36 and 57 to Facility Operating Licenses Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal of March 13, 1981, as supplemented by your April 22, 1981 submittal.

These amendments change the Technical Specifications to incorporate conditions under which either unit's Service Water System nuclear header may be secured for maintenance and modification. We have made some changes to your proposed Technical Specification revisions. These changes have been discussed with and agreed to by members of your staff.

Copies of the Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Original Signed by
 T. A. Ippolito

Thomas A. Ippolito, Chief
 Operating Reactors Branch #2
 Division of Licensing

CP
1

Enclosures:

1. Amendment No. 36 to DPR-71
2. Amendment No. 57 to DPR-62
3. Safety Evaluation
4. Notice

cc w/enclosures:
 See next page

8105140025

F.R. NOTICE
AMENDMENT

ASB
5/7/81
5/4/81

OFFICE	ORB #2	ORB #2	ADPDR	OELD	ORB #2	RSB
SURNAME	JVan Vliet	mjf SNorris	TNovak	<i>K. RYMAN</i>	Tippolito	
DATE	5/4/81	5/4/81	5/4/81	5/4/81	5/6/81	5/1/81

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Docket Nos. 50-325
and 50-324

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Senior Executive Vice President
Carolina Power & Light Company
336 Fayetteville Street
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SURNAME	JVan Vliet	SNorris	TNovak		Tippolito		
DATE	4/ /81	4/ /81	/ /81	/ /81	/ /81	/ /81	



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket file - 2

May 6, 1981

Docket Nos. 50-325
and 50-324

Mr. J. A. Jones
Senior Executive Vice President
Carolina Power & Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment Nos. 36 and 57 to Facility Operating Licenses Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal of March 13, 1981, as supplemented by your April 22, 1981 submittal.

These amendments change the Technical Specifications to incorporate conditions under which either unit's Service Water System nuclear header may be secured for maintenance and modification. We have made some changes to your proposed Technical Specification revisions. These changes have been discussed with and agreed to by members of your staff.

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Sincerely,

Thomas A. Ippolito
Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

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2. Amendment No. 57 to DPR-62
3. Safety Evaluation
4. Notice

cc w/enclosures:
See next page

Mr. J. A. Jones
Carolina Power & Light Company

cc:

Richard E. Jones, Esquire
Carolina Power & Light Company
336 Fayetteville Street
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Southport, North Carolina 28461

Director, Criteria and Standards
Division
Office of Radiation Programs (ANR-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
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Atlanta, Georgia 30308

Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 1057
Southport, North Carolina 28461



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company dated March 13, 1981, as supplemented by letter dated April 22, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 36, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 36

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Remove the following pages and replace with identically numbered pages.

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Add 3/4 10-5
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The underlined pages are changed. Overleaf pages are provided for convenience.

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3/4.7 PLANT SYSTEMS

3/4.7.1 SERVICE WATER SYSTEMS

RESIDUAL HEAT REMOVAL SERVICE WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.1 Two independent Residual Heat Removal Service Water (RHRSW) System subsystems shall be OPERABLE with each subsystem comprised of:

- a. Two pumps, and
- b. An OPERABLE flow path for heat removal capable of taking suction from the intake canal via the service water system and transferring the water through an RHR heat exchanger.

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

- a. With one RHRSW pump inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable; restore the inoperable pump to OPERABLE status within 31 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With one RHRSW subsystem inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable; restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With both RHRSW subsystems inoperable, restore at least one subsystem to OPERABLE status within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.7.1.1 Each residual heat removal service water subsystem shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position, and
- b. At least once per 92 days by verifying that each pump develops a pump ΔP of at least 232 psi at a flow of 4000 gpm measured through the heat exchanger with a minimum suction pressure of 20 psig.

PLANT SYSTEMS

SERVICE WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2 The service water system nuclear header shall be OPERABLE with at least three OPERABLE service water pumps.

APPLICABILITY: CONDITIONS 1, 2, 3, 4 and 5.

ACTION:

a. In CONDITION 1, 2, or 3:

1. With only two service water pumps OPERABLE, restore at least three pumps to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
2. With only one service water pump OPERABLE, restore at least two pumps to OPERABLE status within 72 hours and restore at least three pumps to OPERABLE status within 7 days from the time of the initial loss or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. In Condition 4 or 5:

1. With only one service water pump OPERABLE, restore at least two service water pumps to OPERABLE status within 7 days or declare the Core Spray System, the LPCI System, and the diesel generators inoperable and take the ACTION required by Specifications 3.5.3.1, 3.5.3.2, and 3.8.1.2.
2. With the service water system nuclear header inoperable, operation may continue provided that the Unit No. 2 service water nuclear header is OPERABLE with at least three OPERABLE Unit No. 2 service water pumps; restore the service water system nuclear header to OPERABLE status within 14 days or declare the diesel generators inoperable and take the ACTION required by Specification 3.8.1.2.
3. With the service water system nuclear header inoperable, operation may continue provided that the service water system conventional header is in operation with at least two service water pumps OPERABLE.* Restore the service water system nuclear header to OPERABLE status within 14 days or declare the Core Spray System and LPCI System inoperable and take the ACTION required by Specifications 3.5.3.1 and 3.5.3.2.

*See Special Test Exception 3.10.5

SURVEILLANCE REQUIREMENTS (Continued)

4.7.1.2 The service water system shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual; power operated or automatic) servicing safety related equipment that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 18 months during shutdown, by verifying that each automatic valve servicing safety related equipment actuates to its correct position on the appropriate ECCS actuation test signals.
- c. In CONDITION 4 or 5 with service water system nuclear header inoperable, verify that the service water system conventional header is lined up to supply cooling water to vital ECCS loads and that the Unit 2 nuclear header is lined up to supply cooling water for the diesel generators by verifying that each valve servicing the diesel generators that is not locked open is administratively controlled in the proper position.

SPECIAL TEST EXCEPTIONS

3/4 10.5 PLANT SERVICE WATER

LIMITING CONDITION FOR OPERATION

- 3.10.5 The service water conventional header required to be operating per Specification 3.7.1.2 ACTION b.3 may be removed from operation by stopping the pumps to permit isolating and draining the service water nuclear header for maintenance provided that:
- a. The service water conventional header remains lined up to supply cooling water to the required ECCS loads.
 - b. The draining/maintenance on the service water nuclear header will not affect the service water conventional system or lineup described in a. above.
 - c. Average coolant temperature is $\leq 100^{\circ}\text{F}$ and the heat up rate is $\leq 10^{\circ}\text{F}$ per hour.
 - d. Two dedicated qualified members of the unit operational staff are assigned to initiate the service water conventional header pumps should any of the following occur:
 1. Any event occurs which requires ECCS actuation.
 2. Primary coolant temperature exceeds 180°F .
 3. A loss of off-site power occurs.

APPLICABILITY: CONDITIONS 4 and 5 with the nuclear header inoperable.

ACTION: With the requirements of the above specification not satisfied, as soon as practicable, restore the:

- a. Service water conventional header to operating status per the requirements of Specification 3.7.1.2 ACTION b.3, or
- b. Service water nuclear header to OPERABLE status per Specification 3.7.1.2.

SURVEILLANCE REQUIREMENTS

- 4.10.6 When the service water conventional header is not operating as specified above:
- a. Prior to securing all service water pumps, verify that the service water conventional header is lined up to supply cooling water for ECCS by verifying that each valve servicing safety related equipment that is not locked in the proper position is administratively controlled in the proper position.

SURVEILLANCE REQUIREMENTS

- b. Every four hours, verify that the primary coolant temperature is less than or equal to 180°F.
- c. Prior to securing the service water pumps and at least once per eight hours, verify two-way communications between the Control Room and the Service Water Building.

3/4.10 SPECIAL TEST EXCEPTIONS

BASÉS

3/4.10.1 PRIMARY CONTAINMENT INTEGRITY

The requirement for PRIMARY CONTAINMENT INTEGRITY is removed during the period when open vessel tests are being performed during low power PHYSICS TESTS.

3/4.10.2 ROD SEQUENCE CONTROL SYSTEM

In order to perform the tests required in the Technical Specifications it is necessary to bypass the sequence restraints on control rod movement. The additional surveillance requirements ensure that the specifications on heat generation rates and shutdown margin requirements are not exceeded during the period when these tests are being performed.

3/4.10.3 SHUTDOWN MARGIN DEMONSTRATIONS

Performance of shutdown margin demonstrations with the vessel head removed requires additional restrictions in order to ensure that criticality does not occur. These additional restrictions are specified in this LCO.

3/4.10.4 RECIRCULATION LOOPS

This special test exception permits reactor criticality under no flow conditions and is required to perform certain startup and PHYSICS TESTS while at low THERMAL POWER levels.

3/4.10.5 PLANT SERVICE WATER

This Special Test Exception permits securing the Service Water System conventional header when the nuclear header is out of service and is required to permit flange installation in service water system header cross-connect piping.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company dated March 13, 1981, as supplemented by letter dated April 22, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 57, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 57

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Remove the following pages and replace with identically numbered pages.

IX/X
XI/XII
3/4 7-1/3/4 7-2
Add 3/4 7-2a
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3/4.7 PLANT SYSTEMS

3/4.7.1 SERVICE WATER SYSTEMS

RESIDUAL HEAT REMOVAL SERVICE WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.1 Two independent Residual Heat Removal Service Water (RHRSW) System subsystems shall be OPERABLE with each subsystem comprised of:

- a. Two pumps, and
- b. An OPERABLE flow path for heat removal capable of taking suction from the intake canal via the service water system and transferring the water through an RHR heat exchanger.

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

- a. With one RHRSW pump inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable; restore the inoperable pump to OPERABLE status within 31 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With one RHRSW subsystem inoperable, operation may continue and the provisions of Specification 3.0.4 are not applicable; restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With both RHRSW subsystems inoperable, restore at least one subsystem to OPERABLE status within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.7.1.1 Each residual heat removal service water subsystem shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position, and
- b. At least once per 92 days by verifying that each pump develops a pump ΔP of at least 232 psi at a flow of 4000 gpm measured through the heat exchanger with a minimum suction pressure of 20 psig.

PLANT SYSTEMS

SERVICE WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2 The service water system nuclear header shall be OPERABLE with at least three OPERABLE service water pumps.

APPLICABILITY: CONDITIONS 1, 2, 3, 4 and 5.

ACTION:

a. In CONDITION 1, 2, or 3:

1. With only two service water pumps OPERABLE, restore at least three pumps to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
2. With only one service water pump OPERABLE, restore at least two pumps to OPERABLE status within 72 hours and restore at least three pumps to OPERABLE status within 7 days from the time of the initial loss or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

b. In Condition 4 or 5:

1. With only one service water pump OPERABLE, restore at least two service water pumps to OPERABLE status within 7 days or declare the Core Spray System, the LPCI System, and the diesel generators inoperable and take the ACTION required by Specifications 3.5.3.1, 3.5.3.2, and 3.8.1.2.
2. With the service water system nuclear header inoperable, operation may continue provided that the Unit No. 1- service water nuclear header is OPERABLE with at least three OPERABLE Unit No. 1 service water pumps; restore the service water system nuclear header to OPERABLE status within 14 days or declare the diesel generators inoperable and take the ACTION required by Specification 3.8.1.2.
3. With the service water system nuclear header inoperable, operation may continue provided that the service water system conventional header is in operation with at least two service water pumps OPERABLE.* Restore the service water system nuclear header to OPERABLE status within 14 days or declare the Core Spray System and LPCI System inoperable and take the ACTION required by Specifications 3.5.3.1 and 3.5.3.2.

*See Special Test Exception 3.10.5

SURVEILLANCE REQUIREMENTS (Continued)

4.7.1.2 The service water system shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 18 months during shutdown, by verifying that each automatic valve servicing safety related equipment actuates to its correct position on the appropriate ECCS actuation test signals.
- c. In CONDITION 4 or 5 with service water system nuclear header inoperable, verify that the service water system conventional header is lined up to supply cooling water to vital ECCS loads and that the Unit 1 nuclear header is lined up to supply cooling water for the diesel generators by verifying that each valve servicing the diesel generators that is not locked open is administratively controlled in the proper position.

SPECIAL TEST EXCEPTIONS

3/4 10.5 PLANT SERVICE WATER

LIMITING CONDITION FOR OPERATION

- 3.10.5 The service water conventional header required to be operating per Specification 3.7.1.2 ACTION b.3 may be removed from operation by stopping the pumps to permit isolating and draining the service water nuclear header for maintenance provided that:
- a. The service water conventional header remains lined up to supply cooling water to the required ECCS loads.
 - b. The draining/maintenance on the service water nuclear header will not affect the service water conventional system or lineup described in a. above.
 - c. Average coolant temperature is $\leq 100^{\circ}\text{F}$ and the heat up rate is $\leq 10^{\circ}\text{F}$ per hour.
 - d. Two dedicated qualified members of the unit operational staff are assigned to initiate the service water conventional header pumps should any of the following occur:
 1. Any event occurs which requires ECCS actuation.
 2. Primary coolant temperature exceeds 180°F .
 3. A loss of off-site power occurs.

APPLICABILITY: CONDITIONS 4 and 5 with the nuclear header inoperable.

ACTION: With the requirements of the above specification not satisfied, as soon as practicable, restore the:

- a. Service water conventional header to operating status per the requirements of Specification 3.7.1.2 ACTION b.3, or
- b. Service water nuclear header to OPERABLE status per Specification 3.7.1.2.

SURVEILLANCE REQUIREMENTS

- 4.10.6 When the service water conventional header is not operating as specified above:
- a. Prior to securing all service water pumps, verify that the service water conventional header is lined up to supply cooling water for ECCS by verifying that each valve servicing safety related equipment that is not locked in the proper position is administratively controlled in the proper position.

SURVEILLANCE REQUIREMENTS

- b. Every four hours, verify that the primary coolant temperature is less than or equal to 180°F.
- c. Prior to securing the service water pumps and at least once per eight hours, verify two-way communications between the Control Room and the Service Water Building.

3/4.10 SPECIAL TEST EXCEPTIONS

BASIS

3/4.10.1 PRIMARY CONTAINMENT INTEGRITY

The requirement for PRIMARY CONTAINMENT INTEGRITY is removed during the period when open vessel tests are being performed during low power PHYSICS TESTS.

3/4.10.2 ROD SEQUENCE CONTROL SYSTEM

In order to perform the tests required in the Technical Specifications it is necessary to bypass the sequence restraints on control rod movement. The additional surveillance requirements ensure that the specifications on heat generation rates and shutdown margin requirements are not exceeded during the period when these tests are being performed.

3/4.10.3 SHUTDOWN MARGIN DEMONSTRATIONS

Performance of shutdown margin demonstrations with the vessel head removed requires additional restrictions in order to ensure that criticality does not occur. These additional restrictions are specified in this LCO.

3/4.10.4 RECIRCULATION LOOPS

This special test exception permits reactor criticality under no flow conditions and is required to perform certain startup and PHYSICS TESTS while at low THERMAL POWER levels.

3/4.10.5 PLANT SERVICE WATER

This Special Test Exception permits securing the Service Water System conventional header when the nuclear header is out of service and is required to permit flange installation in service water system header cross-connect piping.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 36 TO FACILITY LICENSE NO. DPR-71 AND
AMENDMENT NO. 57 TO FACILITY LICENSE NO. DPR-62
CAROLINA POWER & LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 AND 50-324

I. INTRODUCTION

By letter dated March 13, 1981, the Carolina Power & Light Company (the licensee) submitted proposed changes to the Technical Specifications appended to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant (BSEP), Unit Nos. 1 and 2. The proposed changes specify the plant and system conditions under which either unit's Service Water System nuclear header may be secured and taken out of service for maintenance and/or modification. Following a review of that submittal, a meeting was held with the licensee to further explain and justify the proposed changes. Based upon the discussion at that meeting, the licensee submitted supplemental information and revised Technical Specifications in a letter dated April 22, 1981.

II. BACKGROUND AND DISCUSSION

Each unit's Service Water System consists of a nuclear header and a conventional header. The nuclear header supplies the normal source of cooling water to the diesel generators and the Reactor Building cooling loads. The safety related loads are: Residual Heat Removal (RHR) service water heat exchangers and pump coolers, RHR pump seal coolers, Emergency Core Cooling System (ECCS) room coolers, and Reactor Building Closed Cooling Water (RBCCW) heat exchangers. The alternate source of cooling water for one unit's diesel generators is supplied by the other unit's Service Water System nuclear header. The alternate source of cooling for either unit's Reactor Building cooling loads is supplied by that unit's conventional header.

The licensee's request for changes to the Technical Specifications that would permit securing either unit's Service Water System nuclear header was submitted as a result of the need to periodically inspect, maintain, and clean the header and header components. The existing Technical Specifications do not establish the plant and system conditions under which the header may be secured. The licensee included in the request a proposed Special Test Exception that would, under certain plant conditions, permit securing the Service Water System conventional header pumps for several

hours while the nuclear header is out of service. This Special Test Exception may be needed to permit nuclear header maintenance if it becomes necessary because of excessive isolation valve leakage to install blank flanges on the conventional header to nuclear header cross-connect piping.

The licensee has determined that the nuclear service water header may be safely secured if:

1. The plant is in Operating Condition 4 (cold shutdown) or Operating Condition 5 (refueling);
2. The other unit's Service Water System nuclear header is operating as the source of cooling water for the diesel generators; and,
3. The affected unit's conventional header is operating as the alternate source of cooling to the Reactor Building loads.

For the Special Test Exception, the licensee has determined that under certain conditions, which include a specified maximum heatup rate and the stationing of dedicated qualified personnel, the conventional header pumps may be safely secured when the nuclear header is inoperable provided that the Service Water System conventional header pumps are restarted if:

1. Any event occurs which requires ECCS actuation;
2. Primary coolant temperature exceeds 180°; or
3. A loss of off-site power occurs.

III. EVALUATION

We have reviewed the licensee's submittals and have determined that the proposed revisions to the Technical Specifications will ensure that for Operating Condition 4 or Operating Condition 5, an adequate source of cooling water will be provided to the Reactor Building cooling loads and the diesel generators. We find that upon incorporation of the proposed Technical Specifications, the ability of the ECCS systems and diesel generators to perform their intended safety functions will not be impaired when the Service Water System nuclear header is taken out of service.

We investigated the possibility of a single active failure resulting in the interruption of cooling water flow to either the diesel generators or the Reactor Building cooling loads. During this investigation we determined that, when in the alternate service water source configuration, if the normally open diesel generator service water supply valve from the opposite

unit was shut or was to fail in the shut position, then service water flow would not be provided to the affected unit's diesel generators. To eliminate the possibility of this occurrence, the licensee agreed, in their April 22, 1981 submittal, to revise the Operating Work Procedures (OWP) to require that the appropriate service water supply valve be locked open when the diesel generators are cooled by the alternate service water source. (Subsequently, it was agreed that since the valves in question are motor operated valves, they would be de-energized in the appropriate position and placed under administrative controls.) We find this to be an acceptable condition.

Our investigation revealed no other instances where a single active failure would prevent adequate service water flow. With the added assurance of the OWP revision agreed to by the licensee, we find that no single active failure could result in inadequate diesel generator cooling or ECCS room and component cooling.

We reviewed the licensee's proposed Special Test Exception and analyzed the amount of time that would be available, under that exception, for blank flange installation or other similar maintenance action. If the average coolant temperature is $\leq 100^{\circ}\text{F}$ at the start, as required by the proposed Technical Specifications, and if the heatup rate is $\leq 10^{\circ}\text{F}$ per hour, as limited by the proposed Technical Specifications, then the conventional header pumps may be secured for up to eight hours before the average coolant temperature reaches 180° and action must be taken.

The licensee estimates that it will take six to eight hours to perform a single blank flange installation. Therefore, we conclude that the subject blank flange installations can be performed in accordance with the proposed Special Test Exception without exceeding the 180°F temperature limit imposed by the Special Test Exception.

During our review of the licensee's proposed Special Test Exception, we questioned whether one operator would be a sufficient level of manning to expeditiously initiate conventional header flow, if necessary. We also questioned the adequacy of the communication system that would exist between the operator and the control room. Based upon these questions, and other questions raised in the March 24, 1981 meeting, the licensee revised and resubmitted the proposed Special Test Exception as part of their April 22, 1981 submittal. The proposed Special Test Exception, as revised, requires two dedicated qualified operators to be stationed and also requires periodic verification of communications.

We find, in summary, that the proposed Special Test Exception as revised is acceptable for use. Minor editorial changes were made to conform the wording to standard Technical Specification language.

IV. ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

V. CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 6, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-325 AND 50-324CAROLINA POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 36 and 57 to Facility Operating License Nos. DPR-71 and DPR-62 issued to Carolina Power & Light Company (the licensee) which revised the Technical Specifications for operation of the Brunswick Steam Electric Plant, Units Nos. 1 and 2 (the facility), located in Brunswick County, North Carolina. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications to incorporate conditions under which either unit's Service Water System nuclear header may be secured for maintenance and modification.

The application for amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of the amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

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For further details with respect to this action, see (1) the application for amendments dated March 13, 1981, as supplemented April 22, 1981, (2) Amendment Nos. 36 and 57 to License Nos. DPR-71 and DPR-62, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Southport-Brunswick County Library, 109 West Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 6th day of May 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing