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Docket No. 50-325

SEPTEMBER 29 1980

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Mr. J. A. Jones
Executive Vice President
Carolina Power & Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment No. 30 to Facility License No. DPR-71 for Brunswick Steam Electric Plant, Unit No. 1. This amendment consists of a change to the Technical Specifications in response to your request dated September 15, 1980.

The amendment changes the Technical Specifications for safety-relief valve settings by extending the effective end date until the completion of the T-quencher modification, rather than applying a specific end date.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 30 to DPR-71
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

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CP F.R. Initial
+ Amendment

OFFICE	ORB #2	ORB #2	AD:OR	ORB #2	OELD
SURNAME	SNorris	JHannon:mjf	TNovak	Tippolito	KARMAK
DATE	9/24/80	9/24/80	9/25/80	9/24/80	9/26/80

Mr. J. A. Jones
Carolina Power & Light Company

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September 29, 1980

cc:

Richard E. Jones, Esquire
Carolina Power & Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

Mr. Fred Tollison
Plant Manager
P. O. Box 458
Southport, North Carolina 28461

George F. Trowbridge, Esquire
Shaw, Pittman, Potts & Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

John J. Burney, Jr., Esquire
Burney, Burney, Sperry & Barefoot
110 North Fifth Avenue
Wilmington, North Carolina 28461

Mr. Franky Thomas, Chairman
Board of Commissioners
P. O. Box 249
Bolivia, North Carolina 28422

Denny McGuire (Ms)
State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, North Carolina 27603

Southport - Brunswick County Library
109 W. Moore Street
Southport, North Carolina 28461

Director, Technical Assessment Division
Office of Radiation Programs (AW-459)
US EPA
Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N. W.
Atlanta, Georgia 30308

Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 1057
Southport, North Carolina 28461



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE


Amendment No. 30
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company dated September 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:
 - (2) The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 30 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas K. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 29, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 30

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following page of the Technical Specifications contained in Appendix A of the above-indicated license with the attached page. The changed area of the revised page is reflected by a marginal line.

Remove

3/4 4-4

Insert

3/4 4-4

REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY/RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.4.2 The safety valve function of all reactor coolant system safety/relief valves shall be OPERABLE with lift settings within $\pm 1\%$ of the following values.*#

- 4 Safety-relief valves @ 1105 psig.
- 4 Safety-relief valves @ 1115 psig.
- 3 Safety-relief valves @ 1125 psig.

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

- a. With the safety valve function of one safety/relief valve inoperable, restore the inoperable safety valve function of the valve to OPERABLE status within 31 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With the safety valve function of two safety/relief valves inoperable, restore the inoperable safety valve function of at least one of the valves to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With the safety valve function of more than two safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.2 The safety valve function of each of the above required safety/relief valves shall be demonstrated OPERABLE by verifying that the bellows on the safety/relief valves have integrity, by instrumentation indication, at least once per 24 hours.

*The lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperature and pressure.

#From Spring, 1980 until the completion of the T-quencher modification, the safety-relief valve lift settings shall be arranged such that each safety-relief valve pair has a minimum nominal lift setting differential of 20 psi and shall be within $\pm 1\%$ of the following values:

- 2 Safety-relief valves @ 1095 psig
- 3 Safety-relief valves @ 1105 psig
- 3 Safety-relief valves @ 1115 psig
- 3 Safety-relief valves @ 1125 psig



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 30 TO FACILITY LICENSE NO. DPR-71

CAROLINA POWER & LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1

DOCKET NO. 50-335

1.0 Introduction

By letter dated September 15, 1980 Carolina Power & Light Company (CP&L or licensee) requested an extension of the end date for temporary setpoints in effect on 3 of the 11 BSEP-1 safety-relief valves (SRVs). This change was necessitated by revisions to the BSEP outage schedule which deferred the T-quencher modification on BSEP-1 to an outage in spring 1982. The staff was advised of the Mark I Containment Program modification schedule by CP&L letter dated September 2, 1980.

2.0 Discussion

Amendment No. 29 to Facility Operating License No. DPR-71 authorized a 20 psi SRV setpoint differential pressure spread for each of the paired SRVs. This change was made to provide added assurance that SRVs with paired discharge headers would not be likely to experience "a pipe organ" response in an overpressure transient, similar to the event that occurred following a BSEP-1 scram on November 20, 1979. (Refer to LER 1-79-107 dated December 20, 1979). The staff concluded that the setpoint spread was an adequate compensatory measure, pending installation of the T-quencher modification and removal of the paired discharge headers.

3.0 Evaluation

The end date of the original TS change was September 1980, as requested by the licensee. Had the end date been keyed to the completion of the T-quencher modification, it would have been equally acceptable.

4.0 Finding

The SRV setpoint spread authorized by Amendment No. 29 to DPR-71 is an acceptable interim measure until such time as the SRV discharge headers are modified under the Long Term Mark I Program. Therefore, the TS change proposed by the licensee is acceptable.

5.0 Environmental Consideration

We have determined that the license amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will

not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

6.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 29, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-325CAROLINA POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 30 to Facility Operating License No. DPR-71, issued to Carolina Power & Light Company which revised Technical Specifications for operation of the Brunswick Steam Electric Plant, Unit No. 1 (the facility) located in Brunswick County, North Carolina. The amendment is effective as of its date of issuance.

The amendment changes the Technical Specifications for safety-relief settings by extending the effective end date until the completion of the T-quencher modification, rather than applying a specific end date.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

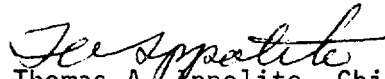
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For further details with respect to this action, see (1) the application for amendment dated September 15, 1980, (2) Amendment No. 30 to License No. DPR-71, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Southport-Brunswick County Library, 109 W. Moore Street, Southport, North Carolina 28461.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 29th day of September 1980.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Appolito, Chief
Operating Reactors Branch #2
Division of Licensing