50-325/324

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 29, 1998

Mr. J. S. Keenan, Vice President
Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

SUBJECT: CORRECTION OF DISCREPANCY IDENTIFIED IN AMENDMENT NO. 203 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 233 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2 (TAC NOS. M97243 AND M97244)

Dear Mr. Keenan:

In a letter dated June 12, 1998, Carolina Power & Light Company (CP&L) identified a discrepancy in Amendment No. 203 to the facility operating license DPR-71 and Amendment No. 233 to the facility operating license DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2 (BSEP 1&2). Those amendments were issued on June 5, 1998, and approved the conversion from the current Technical Specifications (TS) to the improved TS, as contained in Revision 1 of NUREG-1433, "Standard Technical Specifications General Electric Plants, BWR/4."

As issued under those amendments, Table 3.3.6.1-1 on page 3.3-56, "Primary Containment Isolation Instrumentation," Function 4.c for isolation of the reactor core isolation cooling system (RCIC), "RCIC Steam Supply Line Pressure - Low," incorrectly requires the Allowable Value to be $\leq$ 53 psig. The correct Allowable Value is $\geq$ 53 psig.

The erroneous inequality sign was inadvertently introduced in CP&L's November 1, 1996, application for ITS conversion. Table 3.3.6.1-1, Function 4.b, of NUREG-1433 incorrectly lists the Allowable Value of the RCIC Steam Supply Line Pressure - Low function as $\leq$ [60] psig. CP&L properly incorporated the plant-specific value of 53 psig for this function, but did not identify the erroneous inequality sign. The supporting documentation provided by CP&L for the ITS conversion does not request or describe changing the allowable value in the current TS from $\geq$ 53 psig to $\leq$ 53 psig.

As requested by CP&L in the June 12, 1998, letter, the enclosed revised pages to the BSEP 1&2 improved TS are hereby issued to correct the subject discrepancy.

Sincerely,

ORIGINAL SIGNED BY:

David C. Trimble, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-325
and 50-324

Enclosures: As stated

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Table 3.3.6.1-1 (page 3 of 5)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. HPCI System Isolation (continued)					
i. HPCI Equipment Area Temperature —High	1,2,3	2	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 175°F
4. Reactor Core Isolation Cooling (RCIC) System Isolation					
a. RCIC Steam Line Flow —High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 275% rated steam flow
b. RCIC Steam Line Flow —High Time Delay Relay	1,2,3	1	F	SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.9	≥ 4 seconds and ≤ 12 seconds
c. RCIC Steam Supply Line Pressure —Low	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.7	≥ 53 psig
d. RCIC Turbine Exhaust Diaphragm Pressure —High	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 6 psig
e. Drywell Pressure —High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 1.8 psig
f. RCIC Steam Line Area Temperature —High	1,2,3	1	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 175°F
g. RCIC Steam Line Tunnel Ambient Temperature —High	1,2,3	1	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 200°F
h. RCIC Steam Line Tunnel and Area Temperature —High Time Delay	1,2,3	1	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 30 minutes
i. RCIC Steam Line Tunnel Differential Temperature —High	1,2,3	1	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 50°F

(continued)

Table 3.3.6.1-1 (page 3 of 5)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. HPCI System Isolation (continued)					
i. HPCI Equipment Area Temperature —High	1,2,3	2	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 175°F
4. Reactor Core Isolation Cooling (RCIC) System Isolation					
a. RCIC Steam Line Flow —High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 275% rated steam flow
b. RCIC Steam Line Flow —High Time Delay Relay	1,2,3	1	F	SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.9	≥ 4 seconds and ≤ 12 seconds
c. RCIC Steam Supply Line Pressure —Low	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.7	≥ 53 psig
d. RCIC Turbine Exhaust Diaphragm Pressure —High	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 6 psig
e. Drywell Pressure —High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 1.8 psig
f. RCIC Steam Line Area Temperature —High	1,2,3	1	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 175°F
g. RCIC Steam Line Tunnel Ambient Temperature —High	1,2,3	1	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 200°F
h. RCIC Steam Line Tunnel and Area Temperature —High Time Delay	1,2,3	1	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 30 minutes
i. RCIC Steam Line Tunnel Differential Temperature —High	1,2,3	1	F	SR 3.3.6.1.5 SR 3.3.6.1.6 SR 3.3.6.1.7	≤ 50°F

(continued)