Docket No. 50-325

APR 2 1 1977

Carolina Power & Light Company
ATTN: Mr. J. A. Jones, Executive Vice President
Engineering, Construction, and Operations
336 Fayetteville Street
Raleigh, North Carolina 27602

Gentlemen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 3 TO FACILITY OPERATING LICENSE DPR-71 FOR BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

The Nuclear Regulatory Commission has issued the enclosed Amendment to Facility Operating License No. DPR-71 to Carolina Power & Light Company. This amendment provides clarification of the surveillance intervals and tolerances required to comply with the ASME Code Section XI requirements. This clarification has been incorporated into the Appendix A-Prime Specification 4.0.5 of Operating License No. DPR-71. The change in Specification 4.0.5 will provide a clear consistent interpretation of the inservice inspection and testing surveillance intervals and tolerance requirements of the ASME Code Section XI. The specification changes in 4.0.5 will be included in the Standard Technical Specifications for Unit 2 which are currently being prepared by the Nuclear Regulatory Commission.

We have concluded in the enclosed Safety Evaluation that the issuance of this amendment does not involve a significant hazards consideration and there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner.

We expect to transfer the Brunswick licensing responsibility to the Operating Reactors Branch No. 1 by the end of April 1977. Your NRC contact will be C. Trammell, Project Manager, who also has the responsibility for Unit 2.

Const. 1

GD

OFFICE ➤	LWR 4	LWR 4	OELD	LWR 4		
SURNAME ➤	SERVICE/ng	RPowell		SVarga		
DATE ➤	4/8/77	4/8 /77	4/ /77	4/20/77		

A copy of a related Federal Register Notice is also enclosed.

Sincerely,

Original signed by
Steven A. Varga

Steven A. Varga, Chief
Light Water Reactors Branch 4
Division of Project Management

Enclosures:
1. Amendment No. 3
2. Safety Evaluation
3. Federal Register Notice

ccs w/encl:
See next page

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CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Carolina Power & Light Company (the licensee) dated August 18, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, A-Prime, and B as revised through Amendment No. 3, are hereby incorporated in the license. Appendix A shall be effective from the date of issuance of the Unit 1 operating license until the Appendix A-Prime becomes effective on or before the initial criticality of Brunswick Unit 2 following its initial refueling outage. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications as indicated above. The licensee shall inform the Office of Inspection and Enforcement, Region II, of the date that the Appendix A-Prime becomes effective.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION:

Original signed by
Steven A. Varga

Steven A. Varga, Chief
Light Water Reactors Branch No. 4
Division of Project Management

Attachment:
Changes to the Technical
Specifications

Date of Issuance:

APR 2 1 1977

OFFICE ➤						
SURNAME ➤						
DATE ➤						

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:

A. The application for amendment by Carolina Power & Light Company (the licensee) dated August 18, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;

E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and

F. Publication of notice of this amendment is not required since it does not involve a significant hazards consideration nor amendment of a license of the type described in 10 CFR Section 2.106(a)(2).

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2. Accordingly, the license is amendmed by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, A-Prime, and B as revised through Amendment No. , are hereby incorporated in the license. Appendix A shall be effective from the date of issuance of the Unit 1 operating license until the Appendix A-Prime becomes effective on or before the initial criticality of Brunswick Unit 2 following its initial refueling outage. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications as indicated above. The licensee shall inform the Office of Inspection and Enforcement, Region II, of the date that the Appendix A-Prime becomes effective.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION:

Steven A. Varga, Chief
Light Water Reactors Branch No. 4
Division of Project Management

Attachment:
Changes to the Technical
Specifications

Date of Issuance:

STSG 4/20/77 | CMTravell 4-20-77 | A. Schwencer 4-20-77

OFFICE	DPM/LWR-4	DPM/LWR-4	OELD	DPM/LWR-4		
SURNAME	MService:nlg	RPowell	A. Mitchell	SAVarga		
DATE	4/ /77	4/ /77	4/ /77	4/20/77		

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Carolina Power & Light Company (the licensee) dated August 18, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, A-Prime, and B as revised through Amendment No. 3, are hereby incorporated in the license. Appendix A shall be effective from the date of issuance of the Unit 1 operating license until the Appendix A-Prime becomes effective on or before the initial criticality of Brunswick Unit 2 following its initial refueling outage. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications as indicated above. The licensee shall inform the Office of Inspection and Enforcement, Region II, of the date that the Appendix A-Prime becomes effective.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION:

Steven A. Varga, Chief
Light Water Reactors Branch No. 4
Division of Project Management

Attachment:
Changes to the Technical
Specifications

Date of Issuance:

APR 2 1 1977

INSTRUCTION FOR TECHNICAL SPECIFICATION CHANGE TO APPENDIX A-PRIME FOR OPERATING LICENSE DPR-71

Remove pages		and	Replace with pages	
3/4	0-2		3/4	0-2 Mar. 1977
3/4	0-3		3/4	0-3 Mar. 1977
B3/4	0-3		B3/4	0-3 Mar. 1977

b. A total maximum combined interval time for any 3 consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval.

4.0.3 Performance of a Surveillance Requirement within the specified time interval shall constitute compliance with OPERABILITY requirements for a Limiting Condition for Operation and associated ACTION statements unless otherwise required by the specification. Surveillance requirements do not have to be performed on inoperable equipment.

4.0.4 Entry into an OPERATIONAL CONDITION or other specified applicable state shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation have been performed within the applicable surveillance interval or as otherwise specified.

4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2 & 3 components shall be applicable as follows:

a. During the time period:

1. From issuance of the Facility Operating License to the start of facility commercial operation, inservice testing of ASME Code Class 1, 2 & 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code 1974 Edition, and Addenda through Winter 1975 except where specific written relief has been granted by the commission.

2. Following start of facility commercial operation, inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g) (6) (i).

SURVEILLANCE REQUIREMENTS (Continued)

b. Surveillance intervals specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for the inservice inspection and testing activities required by the ASME Boiler and Pressure Vessel Code and applicable Addenda shall be applicable as follows in these Technical Specifications:

ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for inservice inspection and testing activities	Required frequencies for performing inservice inspection and testing activities
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Yearly or annually	At least once per 366 days

c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.

d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.

e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

4.0.5 This specification ensures that inservice inspection of ASME Code Class 1, 2 and 3 components and inservice testing of ASME Code Class 1, 2 and 3 pumps and valves will be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Adenda as required by 10 CFR 50, Section 50.55a. Relief from any of the above requirements has been provided in writing by the Commission and is not a part of these technical specifications.

This specification includes a clarification of the frequencies for performing the inservice inspection and testing activities required by Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda. This clarification is provided to ensure consistency in surveillance intervals throughout these Technical Specifications and to remove any ambiguities relative to the frequencies for performing the required inservice inspection and testing activities.

Under the terms of this specification, the more restrictive requirements of the Technical Specifications take precedence over the ASME Boiler and Pressure Vessel Code and applicable Addenda. For example, the requirements of Specification 4.0.4 to perform surveillance activities prior to entry into an OPERATIONAL MODE or other specified applicability condition takes precedence over the ASME Boiler and Pressure Vessel Code provision which allows pumps to be tested up to one week after return to normal operation. And for example, the Technical Specification definition of OPERABLE does not grant a grace period before a device that is not capable of performing its specified function is declared inoperable and takes precedence over the ASME Boiler and Pressure Vessel Code provision which allows a valve to be incapable of performing its specified function for up to 24 hours before being declared inoperable.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-325

CAROLINA POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY

OPERATING LICENSE DPR-71

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 3 to Facility Operating License No. DPR-71, issued to Carolina Power & Light Company, which revised Technical Specifications for operation of the Brunswick Steam Electric Plant Unit 1 (the facility) located near Southport in Brunswick County, North Carolina. The amendment is effective as of its date of issuance.

The amendment provides a clarification of the tolerance and intervals required for inservice inspection and testing specified in the ASME Code Section XI.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR § 51.5 (d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 18, 1976, (2) Amendment No. 3 to License No. DPR-71, ard (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Brunswick County Library, 109 W. Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 21st day of April, 1977

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Light Water Reactors Branch No. 4
Division of Project Management

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR
REACTOR REGULATION SUPPORTING AMENDMENT NO.
TO LICENSE DPR-71

CAROLINA POWER & LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325

Introduction

In a recent letter dated August 18, 1976, Carolina Power and Light Company requested a tolerance of +25% for the surveillance intervals specified in ASME Code Section XI if no tolerance was specified. Our review of this matter indicated that a clarification of the tolerances to be used for the Section XI surveillance interval was required. We have considered the August 18, 1976 letter from the licensee as a request for a Technical Specification change. In order to provide this clarification, we are providing a Technical Specification change to Specification 4.05 of the Brunswick Unit 1 operating license DPR-71 Appendix A-Prime. Appendix A-Prime of DPR-71 will not become effective until following the refueling of Unit 2 at which time both units will operate in accordance with the standard boiling water technical specifications.

Discussion

The Technical Specification changes to Specification 4.0.5 of Appendix A-Prime were developed by the staff and discussed with the licensee. These changes will provide clarification to tolerance values and test intervals to be used in implementing the surveillance requirements for inservice inspection and testing of ASME Code Class 1,2, and 3 components. This clarification will assist the Office of Inspection and Enforcement personnel in their interpretation of whether the surveillance intervals required by the ASME Code Section XI are being properly implemented.

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This clarification provided in the revised Specification 4.0.5 of the technical specifications will be incorporated into all future standard technical specifications for both BWRs or PWRs.

Evaluation

Our review of the tolerance values permitted for the surveillance intervals specified in ASME Code Section XI for inservice inspection and testing of Class 1, 2, and 3 components indicated a clarification was required to provide a consistent interpretation of the tolerance values to be implemented. The changes in the revised Specification 4.0.5 of the Appendix A-Prime will provide the necessary clarification and be consistent with the Specification 4.0.2. Also, we have included in the Bases of Section 4.0.5 our interpretation of two matters which do not agree with the ASME Code Section XI. These are:

(1) Specification 4.04 of the Technical Specifications requires that surveillance activities must be completed prior to entry into an Operational Mode or other specified applicability condition whereas the ASME Code Section XI would allow pumps which could not be tested during an outage to be tested up to 1 week after return to normal operation. We require that Specification 4.0.4 has precedence.

(2) The definition of Operable in the Technical Specifications does not grant a grace period before a device that is not capable of performing its specified function is declared Inoperable whereas the ASME Code Section XI would permit a valve to be incapable of

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performing its specified function for up to 24 hours before being declared inoperable. We require the Technical Specification definition of Operable and Inoperable shall be used with no grace period allowed.

We conclude that the changes in Specification 4.0.5 of the Appendix A-Prime of Facility Operating License DPR-71 will provide a clarification and consistent interpretation of the surveillance interval requirements of the ASME Code Section XI and are acceptable.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequence of accidents previously considered and does not involve a significant decrease in a safety margin, and the amendment does not involve a significant hazards consideration, (2) there is reasonable

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assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Ray Powell, Project Manager
Light Water Reactors Branch 4
Division of Project Management

Original signed by
Steven A. Varga

Steven A. Varga, Chief
Light Water Reactors Branch 4
Division of Project Management

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