

August 28, 1997

Mr. C. S. Hinnant, Vice President
Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

SUBJECT: ISSUANCE OF AMENDMENT NO. 186 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 217 TO FACILITY OPERATING LICENSE NO. DPR-62 REGARDING REVISED VALUES FOR MINIMUM AND MAXIMUM SUPPRESSION CHAMBER WATER VOLUME - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2 (TAC NOS. M98037 AND M98038)

Dear Mr. Hinnant:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 186 to Facility Operating License No. DPR-71 and Amendment No. 217 to Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated January 15, 1997, and a Unit 1 Bases page correction as requested in your letter dated August 22, 1997.

The amendments revise the minimum and maximum allowed values in Technical Specification 3.6.2.1 for suppression chamber water volume. The amendments correct an error identified by Carolina Power & Light Company in the previous calculation of water volume.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register Notice.

Sincerely,

Original signed by:

David C. Trimble, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-325
and 50-324

Enclosures:

1. Amendment No. 186 to License No. DPR-71
2. Amendment No. 217 to License No. DPR-62
3. Safety Evaluation

cc w/enclosures: See next page
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(Yes/No)	(Yes/No)	Yes/No	Yes/No	

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AMENDMENT NO. 186 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK, UNIT 1
AMENDMENT NO. 217 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK, UNIT 2

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B. Boger

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G. Hill (4)

W. Beckner

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OPA

OC/LFDCB

cc: Brunswick Service List

Mr. C. S. Hinnant
Carolina Power & Light Company

Brunswick Steam Electric Plant
Units 1 and 2

cc:

Mr. William D. Johnson
Vice President and Senior Counsel
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Ms. Karen E. Long
Assistant Attorney General
State of North Carolina
Post Office Box 629
Raleigh, North Carolina 27602

Mr. Jerry W. Jones, Chairman
Brunswick County Board of Commissioners
Post Office Box 249
Bolivia, North Carolina 28422

Mr. Robert P. Gruber
Executive Director
Public Staff - NCUC
Post Office Box 29520
Raleigh, North Carolina 27626-0520

Resident Inspector
U.S. Nuclear Regulatory Commission
8470 River Road
Southport, North Carolina 28461

Mr. W. Levis
Director
Site Operations
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303

Mr. William H. Crowe, Mayor
City of Southport
201 East Moore Street
Southport, North Carolina 28461

Mr. Mel Fry, Acting Director
Division of Radiation Protection
N.C. Department of Environment,
Health and Natural Resources
3825 Barrett Dr.
Raleigh, North Carolina 27609-7721

Mr. Dan E. Summers
Emergency Management Coordinator
New Hanover County Department of
Emergency Management
Post Office Box 1525
Wilmington, North Carolina 28402

Mr. R. P. Lopriore
Plant Manager
Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

Ms. D. B. Alexander
Manager
Performance Evaluation and
Regulatory Affairs
Carolina Power & Light Company
412 S. Wilmington Street
Raleigh, North Carolina 27601

Public Service Commission
State of South Carolina
Post Office Drawer 11649
Columbia, South Carolina 29211

Mr. Milton Shymlock
U. S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303

Mr. K. R. Jury
Manager - Regulatory Affairs
Carolina Power & Light Company
Post Office Box 10429
Southport, NC 28461-0429



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 186
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated January 15, 1997, as supplemented on August 22, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 186, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gordon Edison, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 28, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 186

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 6-9

B3/4 6-3

Insert Pages

3/4 6-9

B3/4 6-3

CONTAINMENT SYSTEMS

3/4.6.2. DEPRESSURIZATION SYSTEMS

SUPPRESSION CHAMBER

LIMITING CONDITION FOR OPERATION

3.6.2.1 The suppression chamber shall be OPERABLE with:

- a. The pool water:
 1. Volume between 86,545 ft³ and 89,843 ft³, equivalent to a level between -27 inches and -31 inches, and a
 2. Maximum average temperature of 95°F during OPERATIONAL CONDITION 1 or 2, except that the maximum average temperature may be permitted to increase to:
 - a) 105°F during testing which adds heat to the suppression chamber.
 - b) 110°F with THERMAL POWER less than or equal to 1% of RATED THERMAL POWER.
 - c) 120°F with the main steam line isolation valves closed following a scram.
- b. Two OPERABLE suppression chamber water temperature instrumentation channels with a minimum of 11 operable RTD inputs per channel.
- c. A total leakage from the drywell to the suppression chamber of less than the equivalent leakage through a 1-inch diameter orifice at a differential pressure of 1 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With the suppression chamber water level outside the above limits, restore the water level to within the limits within 6 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION 1 or 2 with the suppression chamber average water temperature greater than 95°F, restore the average temperature to less than or equal to 95°F within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours, except, as permitted above:

CONTAINMENT SYSTEMS

BASES

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

The specifications of this section ensure that the primary containment pressure will not exceed the calculated pressure of 49 psig during primary system blowdown from full operating pressure.

The pressure suppression pool water provides the heat sink for the reactor primary system energy release following a postulated rupture of the system. The pressure suppression chamber water volume must absorb the associated decay and structural sensible heat released during primary system blowdown from 1045 psig. Since all of the gases in the drywell are purged into the pressure suppression chamber air space during a loss of coolant accident, the pressure of the liquid must not exceed 62 psig, the suppression chamber maximum pressure. The design volume of the suppression chamber, water and air, was obtained by considering that the total volume of reactor coolant to be condensed is discharged to the suppression chamber and that the drywell volume is purged to the suppression chamber.

Using the minimum or maximum water volumes given in the specification, containment pressure during the design basis accident is approximately 49 psig, which is below the design pressure of 62 psig. Maximum water volume of 89,843 ft³ results in a downcomer submergence of 3'4" and the minimum volume of 86,545 ft³ results in a submergence approximately four inches less. The Monticello tests were run with a submerged length of three feet and with complete condensation. Thus, with respect to the downcomer submergence, this specification is adequate. The maximum temperature at the end of the blowdown tested during the Humboldt Bay and Bodega Bay tests was 170°F, and this is conservatively taken to be the limit for complete condensation of the reactor coolant, although condensation would occur for temperatures above 170°F.

When it is necessary to make the suppression chamber inoperable, this shall only be done as provided in Specification 3.5.3.3.

Under full power operation conditions, blowdown from an initial suppression chamber water temperature of 90°F results in a water temperature of approximately 135°F immediately following blowdown, which is below the temperature 170°F used for complete condensation. At this temperature and atmospheric pressure, the available NPSH exceeds that required by both the RHR and core spray pumps; thus, there is no dependency on containment overpressure during the accident injection phase. If both RHR loops are used for containment cooling, there is no dependency on containment overpressure for post-LOCA operations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 217
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated January 15, 1997, as supplemented on August 22, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 217, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gordon Edison, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 28, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 217

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 6-9

B3/4 6-3

Insert Pages

3/4 6-9

B3/4 6-3

CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION SYSTEMS

SUPPRESSION CHAMBER

LIMITING CONDITION FOR OPERATION

3.6.2.1 The suppression chamber shall be OPERABLE with:

- a. The pool water:
 1. Volume between 86,545 ft³ and 89,843 ft³, equivalent to a level between -27 inches and -31 inches, and a
 2. Maximum average temperature of 95°F during OPERATIONAL CONDITION 1 or 2, except that the maximum average temperature may be permitted to increase to:
 - a) 105°F during testing which adds heat to the suppression chamber.
 - b) 110°F with THERMAL POWER less than or equal to 1% of RATED THERMAL POWER.
 - c) 120°F with the main steam line isolation valves closed following a scram.
- b. Two OPERABLE suppression chamber water temperature instrumentation channels with a minimum of 11 operable RTD inputs per channel.
- c. A total leakage from the drywell to the suppression chamber of less than the equivalent leakage through a 1-inch diameter orifice at a differential pressure of 1 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With the suppression chamber water level outside the above limits, restore the water level to within the limits within 6 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION 1 or 2 with the suppression chamber average water temperature greater than 95°F, restore the average temperature to less than or equal to 95°F within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours, except, as permitted above:

CONTAINMENT SYSTEMS

BASES

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

The specifications of this section ensure that the primary containment pressure will not exceed the calculated pressure of 49 psig during primary system blowdown from full operating pressure.

The pressure suppression pool water provides the heat sink for the reactor primary system energy release following a postulated rupture of the system. The pressure suppression chamber water volume must absorb the associated decay and structural sensible heat released during primary system blowdown from 1020 psig. Since all of the gases in the drywell are purged into the pressure suppression chamber air space during a loss of coolant accident, the pressure of the liquid must not exceed 62 psig, the suppression chamber maximum pressure. The design volume of the suppression chamber, water and air, was obtained by considering that the total volume of reactor coolant to be condensed is discharged to the suppression chamber and that the drywell volume is purged to the suppression chamber.

Using the minimum or maximum water volumes given in the specification, containment pressure during the design basis accident is approximately 49 psig, which is below the design pressure of 62 psig. Maximum water volume of 89,843 ft³ results in a downcomer submergence of 3/4" and the minimum volume of 86,545 ft³ results in a submergence approximately four inches less. The Monticello tests were run with a submerged length of three feet and with complete condensation. Thus, with respect to the downcomer submergence, this specification is adequate. The maximum temperature at the end of the blowdown test during the Humboldt Bay and Bodega Bay tests was 170°F, and this is conservatively taken to be the limit for complete condensation of the reactor coolant, although condensation would occur for temperatures above 170°F.

When it is necessary to make the suppression chamber inoperable, this shall only be done as provided in Specification 3.5.3.3.

Under full power operation conditions, blowdown from an initial suppression chamber water temperature of 90°F results in a water temperature of approximately 135°F immediately following blowdown, which is below the temperature 170°F used for complete condensation. At this temperature and atmospheric pressure, the available NPSH exceeds that required by both the RHR and core spray pumps; thus, there is no dependency on containment overpressure during the accident injection phase. If both RHR loops are used for containment cooling, there is no dependency on containment overpressure for post-LOCA operations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 186 TO FACILITY OPERATING LICENSE NO. DPR-71
AND AMENDMENT NO. 217 TO FACILITY OPERATING LICENSE NO. DPR-62
CAROLINA POWER & LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated January 15, 1997, Carolina Power and Light Company (CP&L), the licensee for Brunswick Steam Electric Plant (BSEP), Units 1 and 2, proposed to change Technical Specification (TS) 3.6.2.1.a.1 and corresponding Bases 3/4.6.2 to revise the minimum and maximum values of the pressure suppression pool water volumes from 87,600 ft³ and 89,600 ft³ to 86,545 ft³ and 89,843 ft³. By letter dated August 22, 1997, CP&L requested a correction to the Bases for Unit 1 TS 3/4.6.2, Depressurization and Cooling Systems, reflecting the fact that the recently approved 5 percent uprate in authorized power level (Amendment No. 183 for Unit 1 dated November 1, 1996) increased the primary system operating pressure from the 1020 psig value currently described in the TS Bases to 1045 psig. That amendment was implemented at the conclusion of the fall 1996 Unit 1 refueling outage. The power uprate amendment has not yet been implemented on Unit 2; therefore the corresponding Bases TS will retain the 1020 psig value for primary operating pressure. The information provided in CP&L's August 22, 1997, letter does not affect the conclusions stated in the notice of "Proposed No Significant Hazards Consideration Determination" published in the Federal Register on March 26, 1997 (62 FR 14458).

2.0 DISCUSSION AND EVALUATION

The licensee stated that on October 19, 1996, CP&L completed calculations which indicated that the suppression pool water level operating range of -27" to -31" required by TS 3.6.2.1 was not equivalent to the corresponding water volumes of 89,600 ft³ and 87,600 ft³ delineated in the current BSEP TS. Administrative controls on suppression chamber water level were implemented to ensure the minimum and maximum volumes specified in TS 3.6.2.1.a.1 were maintained. The specific actions taken were reported in Licensee Event Report 1-96-15 and included reanalysis of suppression chamber water volumes. The suppression pool water volume values in the revised calculations were analytically derived based upon a review of plant drawings associated with the suppression chamber and submerged structures. These calculations establish that the -27" and -31" TS limits of suppression pool water level correspond to suppression chamber water volumes of 89,843 ft³ and 86,545 ft³, respectively. CP&L has evaluated the impact of the revised suppression pool water as summarized below.

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Evaluations of the impact of the proposed change to the suppression pool volume limits for postulated (1) loss-of-coolant accident (LOCA), (2) safety relief valve (SRV) blowdown, and (3) anticipated transient without scram (ATWS) events concluded that no adverse impact on containment parameters results from the proposed change to the maximum value of the pressure suppression pool volume. The proposed change to the minimum value does increase the suppression pool water temperature but the increase is not significant and the resulting temperature remains below acceptable limits. Specifically, analyses indicate that the reduction in the minimum suppression pool volume on the pool temperature following the postulated LOCA could result in a peak suppression pool water temperature of 199.5°F, and remains below the design limit of 200°F as specified in TS 5.2.2.b. The slight increase in the suppression pool water temperature associated with LOCA would also result in a slight reduction in the available net positive suction head (NPSH) for the residual heat removal (RHR) and core spray pumps. However, adequate NPSH would be maintained throughout the postulated design basis LOCA.

The evaluation indicated that the impact of the proposed water volume limit reduction on ATWS events is a small increase in the suppression pool water temperature to 167°F which remains well below the specified limit of 190°F for ATWS events. Reduced water volume also results in a slight increase in the peak bulk and local pool temperature for SRV blowdown events to 187.1°F and 198.1°F, which remains within acceptable limits of 200°F and 203°F, respectively.

The analyses evaluated the potential impact of the proposed change to the suppression pool water volume limits on the SRV line loads, SRV discharge line reflood height, wetwell pressurization, suppression pool swell loads, vent thrust loads, and condensation oscillation and chugging loads. The evaluation concluded that the change in suppression pool water volume limits has no adverse impact on these parameters because the bounding suppression pool levels remain unchanged.

The analyses also concluded that the reduction in the minimum suppression pool water volume could slightly increase the peak suppression pool temperature (less than 0.4°F) when an alternate to the RHR shutdown cooling function is used to reach shutdown conditions. The potential increase in peak suppression pool temperature has a negligible impact on the time required by BSEP TS to reach cold shutdown.

The licensee stated that the BSEP analyses related to an Appendix R (10 CFR Part 50) fire event were reviewed to determine the impact of the suppression pool water volume limit change. These analyses indicate that the peak suppression pool water temperature would increase slightly as a result of the proposed change; however, the peak suppression pool water temperature of 186.4°F remains below the suppression pool design limit of 200°F. Also, the resulting peak temperature for the station blackout (SBO) event, using assumptions and methodology consistent with the SBO Safety Evaluation for BSEP, is 198.8°F. This value is below the 200°F acceptance criterion stated in an NRC Safety Evaluation (SE) for SBO, "Station Blackout Evaluation - Brunswick Steam Electric Plant, Units 1 and 2," dated October 4, 1990. A more in-depth discussion of the revised SBO analysis including the assumptions used

was provided in a CP&L letter to NRC dated December 23, 1996, and was found acceptable in an NRC letter to CP&L dated March 18, 1997.

In summary, the staff has reviewed the licensee's submittal as discussed above and finds the proposed TS change acceptable because the impact of the proposed change to the minimum and maximum suppression pool volume limits on suppression pool temperature and pressure following design basis LOCA, SRV blowdown, ATWS, Appendix R fire, and SBO events does not cause accident parameters to exceed acceptable values. The increase in the suppression pool temperature that could result from the proposed change is not significant. Based on the above evaluation, the staff finds the proposed change to TS 3.6.2.1.a.1 and the corresponding Bases 3/4.6.2 to revise the minimum and maximum values of the suppression pool water volumes from 87,600 ft³ and 89,600 ft³ to 86,545 ft³ and 89,843 ft³ acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (62 FR 14458). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Goel

Date: August 28, 1997