May 28, 1998

Mr. C. S. Hinnant, Vice President
Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

SUBJECT: ISSUANCE OF AMENDMENT NO. 197 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 227 TO FACILITY OPERATING LICENSE NO. DPR-62 REVISING TECHNICAL SPECIFICATION (TS) MODIFYING INSTRUMENTATION ALLOWABLE VALUES - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2 (TAC NOS. MA0174 AND MA0175)

Dear Mr. Hinnant:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 197 to Facility Operating License No. DPR-71 and Amendment No. 227 to Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated October 28, 1997.

The amendments revise certain instrumentation allowable values in the Technical Specifications.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register Notice.

Sincerely,

Original signed by:

David C. Trimble, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-325
and 50-324

Enclosures:
1. Amendment No. 197 to License No. DPR-71
2. Amendment No. 227 to License No. DPR-62
3. Safety Evaluation

cc w/enclosures: See next page

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PM:PDII-1	LA:PDII-1	OGC	APD:PDII-1	
DTrimble	EDunnington	RBachmann	PTKuo	
5/13/98	5/13/98	5/14/98	5/28/98	
Yes/No	Yes/No	Yes/No	Yes/No	

OFFICIAL RECORD COPY

cc:

Mr. William D. Johnson
Vice President and Senior Counsel
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Mr. Jerry W. Jones, Chairman
Brunswick County Board of Commissioners
Post Office Box 249
Bolivia, North Carolina 28422

Resident Inspector
U.S. Nuclear Regulatory Commission
8470 River Road
Southport, North Carolina 28461

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303

Mr. Mel Fry, Director
Division of Radiation Protection
N.C. Department of Environment
and Natural Resources
3825 Barrett Dr.
Raleigh, North Carolina 27609-7721

Mr. J. J. Lyash
Plant Manager
Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

Public Service Commission
State of South Carolina
Post Office Drawer 11649
Columbia, South Carolina 29211

Mr. Milton Shymlock
U. S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303

Ms. Karen E. Long
Assistant Attorney General
State of North Carolina
Post Office Box 629
Raleigh, North Carolina 27602

Mr. Robert P. Gruber
Executive Director
Public Staff - NCUC
Post Office Box 29520
Raleigh, North Carolina 27626-0520

Director
Site Operations
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

Mr. William H. Crowe, Mayor
City of Southport
201 East Moore Street
Southport, North Carolina 28461

Mr. Dan E. Summers
Emergency Management Coordinator
New Hanover County Department of
Emergency Management
Post Office Box 1525
Wilmington, North Carolina 28402

Ms. D. B. Alexander
Manager
Performance Evaluation and
Regulatory Affairs CPB 9
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602-1551

Mr. K. R. Jury
Manager - Regulatory Affairs
Carolina Power & Light Company
Post Office Box 10429
Southport, NC 28461-0429

AMENDMENT NO. 197 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK, UNIT 1 AND AMENDMENT NO. 227 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK, UNIT 2

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cc: Brunswick Service List

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 197
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated October 28, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 197 , are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Pao-Tsin Kuo, Acting Director for
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical Specifications

Date of Issuance: May 28, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 197

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages	Insert Pages
2-4	2-4
3/4 3-19	3/4 3-19
3/4 3-39	3/4 3-39
3/4 3-40	3/4 3-40
3/4 3-90	3/4 3-90
3/4 3-95	3/4 3-95

TABLE 2.2.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUES
1. Intermediate Range Monitor, Neutron Flux - High[a]	≤ 120 divisions of full scale	≤ 120 divisions of full scale
2. Average Power Range Monitor		
a. Neutron Flux - High, 15%[b]	≤ 15% of RATED THERMAL POWER	≤ 15% of RATED THERMAL POWER
b. Flow-Biased Simulated Thermal Power - High[c][d]	≤ (0.66W + 59.6%) with a maximum ≤ 113.6% of RATED THERMAL POWER	≤ (0.66W + 61%) with a maximum ≤ 115.3% of RATED THERMAL POWER
c. Fixed Neutron Flux - High[d]	≤ 116.3% of RATED THERMAL POWER	≤ 118.5% of RATED THERMAL POWER
3. Reactor Vessel Steam Dome Pressure - High	≤ 1067.9 psig	≤ 1077 psig
4. Reactor Vessel Water Level - Low, Level 1	≥ +153.2 inches[g]	≥ +153 inches[g]
5. Main Steam Line Isolation Valve - Closure[e]	≤ 10% closed	≤ 10% closed
6. (Deleted)		
7. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
8. Scram Discharge Volume Water Level - High	≤ 109 gallons	≤ 109 gallons
9. Turbine Stop Valve - Closure[f]	≤ 10% closed	≤ 10% closed
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low[f]	≥ 500 psig	≥ 500 psig

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
2. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Building Exhaust Radiation - High	≤ 11 mr/hr	≤ 11 mr/hr
b. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
c. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[a]	≥ + 101 inches[a]
3. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. Δ Flow - High	≤ 73 gal/min	≤ 73 gal/min
b. Area Temperature - High	≤ 150°F	≤ 150°F
c. Area Ventilation Δ Temperature - High	≤ 50°F	≤ 50°F
d. SLCS Initiation	NA	NA
e. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[a]	≥ + 101 inches[a]
f. Δ Flow - High - Time Delay	≤ 30 minutes	≤ 30 minutes
g. Piping Outside RWCU Rooms Area Temperature - High	≤ 120°F	≤ 120°F

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. CORE SPRAY SYSTEM		
a. Reactor Vessel Water Level - Low, Level 3	≥ + 14.1 inches[(b)]	≥ + 13 inches[(b)]
b. Reactor Steam Dome Pressure - Low	≥ 406.7 psig	≥ 402 psig
c. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
d. Time Delay-Relay	14 ≤ t ≤ 16 secs	14 ≤ t ≤ 16 secs
e. Bus Power Monitor	NA	NA
2. LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM		
a. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
b. Reactor Vessel Water Level - Low, Level 3	≥ + 14.1 inches[(b)]	≥ + 13 inches[(b)]
c. Reactor Vessel Shroud Level	≥ - 53 inches[(b)]	≥ - 53 inches[(b)]
d. Reactor Steam Dome Pressure - Low		
1. RHR Pump Start and LCPI Valve Actuation	≥ 406.7 psig	≥ 402 psig
2. Recirculation Pump Discharge Valve Actuation	≥ 306.7 psig	≥ 302 psig
e. RHR Pump Start - Time Delay Relay	9 ≤ t ≤ 11 seconds	9 ≤ t ≤ 11 seconds
f. Bus Power Monitor	NA	NA

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
3. HIGH PRESSURE COOLANT INJECTION SYSTEM		
a. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[b]	≥ + 101 inches[b]
b. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
c. Condensate Storage Tank Level - Low	≥ 23 feet 4 inches	≥ 23 feet 4 inches
d. Suppression Chamber Water Level - High	≤ -2 feet[c]	≤ -2 feet[c]
e. Bus Power Monitor	NA	NA
4. AUTOMATIC DEPRESSURIZATION SYSTEM		
a. ADS Inhibit Switch	NA	NA
b. Reactor Vessel Water Level - Low, Level 3	≥ + 14.1 inches[b]	≥ + 13 inches[b]
c. Reactor Vessel Water Level - Low, Level 1	≥ + 153.2 inches[b]	≥ + 153 inches[b]
d. ADS Timer	≤ 83 seconds	≤ 108 seconds
e. Core Spray Pump Discharge Pressure - High	≥ 112.1 psig	≥ 102 psig
f. RHR (LPCI MODE) Pump Discharge Pressure - High	≥ 111.1 psig	≥ 102 psig
g. Bus Power Monitor	NA	NA

TABLE 3.3.6.1-2

ATWS RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[a]	≥ + 101 inches[a]
2. Reactor Vessel Pressure - High	≤ 1137.8 psig	≤ 1147 psig

[a] Vessel water levels refer to REFERENCE LEVEL ZERO.

TABLE 3.3.7-2

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
1. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[a]	≥ + 101 inches[a]
2. Reactor Vessel Water Level - High	≤ + 206.8 inches[a]	≤ + 207 inches[a]
3. Condensate Storage Tank Level - Low	≥ 23 feet 0 inches	≥ 23 feet 0 inches

(a) Vessel water levels refer to REFERENCE LEVEL ZERO.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 227
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated October 28, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 227 , are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

H E Edison for

Pao-Tsin Kuo, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 28, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 227

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Replace the following pages of Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages	Insert Pages
2-4	2-4
3/4 3-18	3/4 3-18
3/4 3-19	3/4 3-19
3/4 3-39	3/4 3-39
3/4 3-40	3/4 3-40
3/4 3-91	3/4 3-91
3/4 3-102	3/4 3-102

TABLE 2.2.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUES
1. Intermediate Range Monitor, Neutron Flux - High[a]	≤ 120 divisions of full scale	≤ 120 divisions of full scale
2. Average Power Range Monitor		
a. Neutron Flux - High, 15%[b]	≤ 15% of RATED THERMAL POWER	≤ 15% of RATED THERMAL POWER
b. Flow-Biased Simulated Thermal Power - High[c][d]	≤ (0.66W + 59.6%) with a maximum ≤ 113.6% of RATED THERMAL POWER	≤ (0.66W + 61%) with a maximum ≤ 115.3% of RATED THERMAL POWER
c. Fixed Neutron Flux - High[d]	≤ 116.3% of RATED THERMAL POWER	≤ 118.5% of RATED THERMAL POWER
3. Reactor Vessel Steam Dome Pressure - High	≤ 1067.9 psig	≤ 1077 psig
4. Reactor Vessel Water Level - Low, Level 1	≥ +153.2 inches[g]	≥ +153 inches[g]
5. Main Steam Line Isolation Valve - Closure[e]	≤ 10% closed	≤ 10% closed
6. (Deleted)		
7. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
8. Scram Discharge Volume Water Level - High	≤ 109 gallons	≤ 109 gallons
9. Turbine Stop Valve - Closure[f]	≤ 10% closed	≤ 10% closed
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low[f]	≥ 500 psig	≥ 500 psig

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level -		
1. Low, Level 1	≥ + 153.2 inches[a]	≥ + 153 inches[a]
2. Low, Level 3	≥ + 14.1 inches[a]	≥ + 13 inches[a]
b. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
c. Main Steam Line		
1. (Deleted)		
2. Pressure - Low	≥ 825 psig	≥ 825 psig
3. Flow - High	≤ 137% of rated flow	≤ 138% of rated flow
4. Flow - High	≤ 30% of rated flow	≤ 33% of rated flow
d. Main Steam Line Tunnel Temperature - High	≤ 200°F	≤ 200°F
e. Condenser Vacuum - Low	≥ 7.6 inches Hg vacuum	≥ 7.5 inches Hg vacuum
f. Turbine Building Area Temperature - High	≤ 200°F	≤ 200°F
g. Main Stack Radiation - High	(b)	(b)
h. Reactor Building Exhaust Radiation - High	≤ 11 mr/hr	≤ 11 mr/hr

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
2. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Building Exhaust Radiation - High	≤ 11 mr/hr	≤ 11 mr/hr
b. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
c. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[a]	≥ + 101 inches[a]
3. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. Δ Flow - High	≤ 73 gal/min	≤ 73 gal/min
b. Area Temperature - High	≤ 150°F	≤ 150°F
c. Area Ventilation Temperature Δ Temp - High	≤ 50°F	≤ 50°F
d. SLCS Initiation	NA	NA
e. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[a]	≥ + 101 inches[a]
f. Δ Flow - High - Time Delay	≤ 30 minutes	≤ 30 minutes
g. Piping Outside RWCU Rooms Area Temperature - High	≤ 120°F	≤ 120°

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. CORE SPRAY SYSTEM		
a. Reactor Vessel Water Level - Low, Level 3	≥ + 14.1 inches(b)	≥ + 13 inches(b)
b. Reactor Steam Dome Pressure - Low	≥ 406.7 psig	≥ 402 psig
c. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
d. Time Delay-Relay	14 ≤ t ≤ 16 secs	14 ≤ t ≤ 16 secs
e. Bus Power Monitor	NA	NA
2. LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM		
a. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
b. Reactor Vessel Water Level - Low, Level 3	≥ + 14.1 inches(b)	≥ + 13 inches(b)
c. Reactor Vessel Shroud Level	≥ - 53 inches(b)	≥ - 53 inches(b)
d. Reactor Steam Dome Pressure - Low		
1. RHR Pump Start and LCPI Valve Actuation	≥ 406.7 psig	≥ 402 psig
2. Recirculation Pump Discharge Valve Actuation	≥ 306.7 psig	≥ 302 psig
e. RHR Pump Start - Time Delay Relay	9 ≤ t ≤ 11 seconds	9 ≤ t ≤ 11 seconds
f. Bus Power Monitor	NA	NA

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
3. HIGH PRESSURE COOLANT INJECTION SYSTEM		
a. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[b]	≥ + 101 inches[b]
b. Drywell Pressure - High	≤ 2 psig	≤ 2 psig
c. Condensate Storage Tank Level - Low	≥ 23 feet 4 inches	≥ 23 feet 4 inches
d. Suppression Chamber Water Level - High	≤ -2 feet[c]	≤ -2 feet[c]
e. Bus Power Monitor	NA	NA
4. AUTOMATIC DEPRESSURIZATION SYSTEM		
a. ADS Inhibit Switch	NA	NA
b. Reactor Vessel Water Level - Low, Level 3	≥ + 14.1 inches[b]	≥ + 13 inches[b]
c. Reactor Vessel Water Level - Low, Level 1	≥ + 153.2 inches[b]	≥ + 153 inches[b]
d. ADS Timer	≤ 83 seconds	≤ 108 seconds
e. Core Spray Pump Discharge Pressure - High	≥ 112.1 psig	≥ 102 psig
f. RHR (LPCI MODE) Pump Discharge Pressure - High	≥ 111.1 psig	≥ 102 psig
g. Bus Power Monitor	NA	NA

TABLE 3.3.6.1-2

ATWS RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[a]	≥ + 101 inches[a]
2. Reactor Vessel Pressure - High	≤ 1137.8 psig	≤ 1147 psig

[a] Vessel water levels refer to REFERENCE LEVEL ZERO.

TABLE 3.3.7-2

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
1. Reactor Vessel Water Level - Low, Level 2	≥ + 104.1 inches[a]	≥ + 101 inches[a]
2. Reactor Vessel Water Level - High	≤ +206.8 inches[a]	≤ +207 inches[a]
3. Condensate Storage Tank Level - Low	≥ 23 feet 0 inches	≥ 23 feet 0 inches

(a) Vessel water levels refer to REFERENCE LEVEL ZERO.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

ALLOWABLE VALUE CHANGES LICENSE AMENDMENT

CAROLINA POWER AND LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT UNIT NOS. 1 AND 2

DOCKET NOS. 50-325 AND 50-334

1.0 INTRODUCTION

By letter dated October 28, 1997, Carolina Power and Light Company (CP&L) requested an amendment to revise certain instrumentation allowable values in the Technical Specifications (TS) for the Brunswick Steam Electric Plant Unit Nos. 1 and 2.

The licensee is in the process of converting the current Technical Specifications (CTS) to the improved Standard Technical Specifications format. In support of this effort, the licensee has proposed the revision of certain instrumentation allowable values contained in the CTS. The CTS allowable values are based on uncertainties associated with the trip unit portion of the instrumentation circuitry. The proposed allowable values are based on uncertainties associated with the entire loop of the instrumentation circuitry (trip unit and sensor) and were calculated in accordance with the licensee's setpoint methodology described in Design Guide (DG) VIII.0050, "Instrument Setpoints." The staff has previously reviewed and accepted this setpoint methodology.

2.0 EVALUATION

The following TS changes have been proposed by the licensee:

1. Average Power Range Monitor, Fixed Neutron Flux - High
   (Table 2.2.1-1.2c, page 2-4)
   Change Allowable Value from ≤118% of RATED THERMAL POWER to ≤118.5% of RATED THERMAL POWER.

2. Reactor Vessel Steam Dome Pressure - High
   (Table 2.2.1-1.3, page 2-4)
   Change Allowable Value from ≤1070 psig to ≤1077 psig.

3. Primary Containment Isolation, Main Steam Line Flow - High
   (Table 3.3.2-2.1.c.4, page 3/4 3-18) Unit 2 only
   Change Allowable Value from ≤32% of rated flow to ≤33% of rated flow.

4. Secondary Containment Isolation, Reactor Vessel Water Level - Low, Level 2
(Table 3.3.2-2.2.c, page 3/4)
Change Allowable Value from ≥+103 inches to ≥+101 inches.

5. Reactor Water Cleanup System Isolation, Reactor Vessel Water Level - Low, Level 2
(Table 3.3.2-2.3.e, page 3/4 3-19)
Change Allowable Value from ≥+103 inches to ≥+101 inches.

6. Core Spray System, Reactor Steam Dome Pressure - Low
(Table 3.3.3-2.1.b, page 3/4 3-39)
Change Allowable Value from ≥404 psig to≥402 psig

7. Low Pressure Coolant Injection Mode of Residual Heat Removal System, Reactor Steam Dome Pressure - Low (Residual Heat Removal Pump Start and Low Pressure Coolant Injection Valve Actuation)
(Table 3.3.3-2.2.d.1, page 3/43-39)
Change Allowable Value from ≥404 psig to ≥402 psig.

8. Low Pressure Coolant Injection Mode of Residual Heat Removal System, Reactor Steam Dome Pressure - Low (Recirculation Pump Discharge Valve Actuation)
(Table 3.3.3-2.2.d.2, page 3/4 3-39)
Change Allowable Value from ≥304 psig to ≥302 psig.

9. High Pressure Coolant Injection System, Reactor Vessel Water Level - Low, Level 2
(Table 3.3.3-2.3.a, page 3/4)
Change Allowable Value from ≥+103 inches to ≥+101 inches.

10. Anticipated Transient Without Scram - Recirculation Pump Trip System Instrumentation Setpoints, Reactor Vessel Water Level - Low, Level 2
(Table 3.3.6.1-2.1, page 3/4 3-90)
Change Allowable Value from ≥+103 inches to ≥+101 inches.

11. Anticipated Transient Without Scram - Recirculation Pump Trip System Instrumentation Setpoints, Reactor Vessel Pressure - High
(Table 3.3.6.1-2.2, page 3/4 3-90)
Change Allowable Value from ≤1143 psig to ≤1147 psig.

12. Reactor Core Isolation Cooling System Actuation Instrumentation Setpoints, Reactor Vessel Water Level - Low, Level 2
(Table 3.3.7-2.1, page 3/4 3-95)
Change Allowable Value from ≥+103 inches to ≥+101 inches.

The proposed allowable values were calculated by applying calibration-based errors to the trip setpoints, thereby establishing an operability limit associated with the entire loop of each instrumentation function. The proposed allowable value changes are within the analytical limit for each function and do not affect the existing margins between operating conditions and reactor trip setpoints. Therefore, the proposed allowable value changes are acceptable.

Additionally, based on the above evaluation, the staff concludes that the proposed instrumentation allowable value changes incorporated in the TS are consistent with the licensee's setpoint methodology and, therefore, are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (62 FR 68304). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Barry S. Marcus

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