

May 29, 1998

Mr. C. S. Hinnant, Vice President
Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

SUBJECT: ISSUANCE OF AMENDMENT NO. 198 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 228 TO FACILITY OPERATING LICENSE NO. DPR-62 REGARDING MODIFICATION OF THE SURVEILLANCE REQUIREMENTS FOR THE ELECTRICAL PROTECTIVE ASSEMBLIES IN THE REACTOR PROTECTION SYSTEM - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2 (BSEP 95-0561) (TAC NOS. M95417 AND M95418)

Dear Mr. Hinnant:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 198 to Facility Operating License No. DPR-71 and Amendment No. 228 to Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant, Units 1 and 2. The amendments change the Technical Specifications in response to your submittal dated November 15, 1995.

The amendments modify the Technical Specifications Surveillance Requirements for the Electrical Protective Assemblies in the Reactor Protection System.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register Notice.

Sincerely,
Original signed by:
David C. Trimble, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-325
and 50-324

Enclosures:

1. Amendment No. 198 to License No. DPR-71
2. Amendment No. 228 to License No. DPR-62
3. Safety Evaluation

cc w/enclosures: See next page

5037

DFOH

FILENAME - G:\BRUNSWIC\BR95417.AMD

PM:PDII-1	LA:PDII-1	BC:HICB	BC:TSB	OGC	APD:PDI I-1	
DTrimble	EDunnington	JWermiel	F.H. Reortant Per W.D. Beal	R. Bachmann	PTKuo	
5114 198	5129 198	5112 198	5114 198	5120 198	5129 198	
(Yes/No)	(Yes/No)			Yes/No	Yes/No	

OFFICIAL RECORD COPY

9806020270 980529
PDR ADOCK 05000324
P PDR

REG FILE IDENTIFIER COPY

AMENDMENT NO. 198 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK,
UNIT 1 AMENDMENT NO. 228 TO FACILITY OPERATING LICENSE NO. DPR-62 -
BRUNSWICK, UNIT 2

DISTRIBUTION:

Docket File

PUBLIC

PDII-1 Reading File

J. Zwolinski

OGC

G. Hill (4)

J. Wermiel

ACRS

OPA

OC/LFDCB

L. Plisco, RII

cc: Brunswick Service List

Mr. C. S. Hinnant
Carolina Power & Light Company

Brunswick Steam Electric Plant
Units 1 and 2

cc:

Mr. William D. Johnson
Vice President and Senior Counsel
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Ms. Karen E. Long
Assistant Attorney General
State of North Carolina
Post Office Box 629
Raleigh, North Carolina 27602

Mr. Jerry W. Jones, Chairman
Brunswick County Board of Commissioners
Post Office Box 249
Bolivia, North Carolina 28422

Mr. Robert P. Gruber
Executive Director
Public Staff - NCUC
Post Office Box 29520
Raleigh, North Carolina 27626-0520

Resident Inspector
U.S. Nuclear Regulatory Commission
8470 River Road
Southport, North Carolina 28461

Director
Site Operations
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303

Mr. William H. Crowe, Mayor
City of Southport
201 East Moore Street
Southport, North Carolina 28461

Mr. Mel Fry, Director
Division of Radiation Protection
N.C. Department of Environment
and Natural Resources
3825 Barrett Dr.
Raleigh, North Carolina 27609-7721

Mr. Dan E. Summers
Emergency Management Coordinator
New Hanover County Department of
Emergency Management
Post Office Box 1525
Wilmington, North Carolina 28402

Mr. J. J. Lyash
Plant Manager
Carolina Power & Light Company
Brunswick Steam Electric Plant
Post Office Box 10429
Southport, North Carolina 28461

Ms. D. B. Alexander
Manager
Performance Evaluation and
Regulatory Affairs CPB 9
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602-1551

Public Service Commission
State of South Carolina
Post Office Drawer 11649
Columbia, South Carolina 29211

Mr. Milton Shymlock
U. S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303

Mr. K. R. Jury
Manager - Regulatory Affairs
Carolina Power & Light Company
Post Office Box 10429
Southport, NC 28461-0429



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 198
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated November 15, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

9806020276 980529
PDR ADOCK 05000324
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 198 , are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Pao-Tsin Kuo, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 29, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 198

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

XII

3/4 8-15

--

Insert Pages

XII

3/4 8-15

B 3/4 8-2

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.7 PLANT SYSTEMS (Continued)</u>	
3/4.7.3 FLOOD PROTECTION.....	B 3/4 7-1h
3/4.7.4 REACTOR CORE ISOLATION COOLING SYSTEM.....	B 3/4 7-1h
3/4.7.5 SNUBBERS.....	B 3/4 7-2
3/4.7.6 SEALED SOURCE CONTAMINATION.....	B 3/4 7-3
3/4.7.7 FIRE SUPPRESSION SYSTEMS (Deleted).....	B 3/4 7-4
3/4.7.8 FIRE BARRIER PENETRATIONS (Deleted).....	B 3/4 7-5
<u>3/4.8 ELECTRICAL POWER SYSTEMS.....</u>	B 3/4 8-1
3/4.8.2.5 REACTOR PROTECTION SYSTEM ELECTRIC POWER MONITORING.....	B 3/4 8-2
<u>3/4.9 REFUELING OPERATIONS</u>	
3/4.9.1 REACTOR MODE SWITCH.....	B 3/4 9-1
3/4.9.2 INSTRUMENTATION.....	B 3/4 9-1
3/4.9.3 CONTROL ROD POSITION.....	B 3/4 9-1
3/4.9.4 DECAY TIME.....	B 3/4 9-1
3/4.9.5 COMMUNICATIONS.....	B 3/4 9-1
3/4.9.6 CRANE AND HOIST OPERABILITY.....	B 3/4 9-2
3/4.9.7 CRANE TRAVEL-SPENT FUEL STORAGE POOL.....	B 3/4 9-2
3/4.9.8 WATER LEVEL-REACTOR VESSEL, and	
3/4.9.9 WATER LEVEL-REACTOR FUEL STORAGE POOL.....	B 3/4 9-2
3/4.9.10 CONTROL ROD REMOVAL.....	B 3/4 9-2
<u>3/4.10 SPECIAL TEST EXCEPTIONS</u>	
3/4.10.1 PRIMARY CONTAINMENT INTEGRITY.....	B 3/4 10-1
3/4.10.2 ROD SEQUENCE CONTROL SYSTEM (DELETED).....	B 3/4 10-1
3/4.10.3 SHUTDOWN MARGIN DEMONSTRATIONS.....	B 3/4 10-1
3/4.10.4 RECIRCULATION LOOPS.....	B 3/4 10-1
3/4.10.5 PLANT SERVICE WATER.....	B 3/4 10-1

ELECTRICAL POWER SYSTEMS

REACTOR PROTECTION SYSTEM ELECTRIC POWER MONITORING

LIMITING CONDITION FOR OPERATION

3.8.2.5 Two RPS electric power monitoring channels for each inservice RPS MG set or alternate source shall be OPERABLE.

APPLICABILITY: Whenever the respective power supply is supplying power to a RPS bus.

ACTION:

- a. With one RPS electric power monitoring channel for an inservice RPS MG set or alternate power supply inoperable, restore the inoperable channel to OPERABLE status within 72 hours or remove the associated RPS MG set or alternate power supply from service.
- b. With both RPS electric power monitoring channels for an inservice RPS MG set or alternate power supply inoperable, restore at least one to OPERABLE status within 30 minutes or remove the associated RPS MG set or alternate power supply from service.

SURVEILLANCE REQUIREMENTS

4.8.2.5 The above specified RPS power monitoring system instrumentation shall be determined OPERABLE:

- a. By performance of a CHANNEL FUNCTIONAL TEST each time the plant is in COLD SHUTDOWN for a period of more than 24 hours, unless performed in the previous 6 months, and
- b. At least once per 18 months by demonstrating the OPERABILITY of over-voltage, under-voltage, and under-frequency protective instrumentation by performance of a CHANNEL CALIBRATION including simulated automatic actuation of the protective relays, tripping logic, and output circuit breakers and verifying the following setpoints:

	<u>RPS MG SET</u>	<u>ALTERNATE SOURCE</u>
1. Over-voltage	≤ 129 VAC	≤ 132 VAC
2. Under-voltage	≥ 105 VAC	≥ 108 VAC
3. Under-frequency	≥ 57 Hz	≥ 57 Hz

3/4.8 ELECTRICAL POWER SYSTEMS

BASES

3/4.8.2.5 REACTOR PROTECTION SYSTEM ELECTRIC POWER MONITORING

The CHANNEL FUNCTIONAL TEST is only required to be performed while the plant is in a condition in which the loss of the RPS bus will not jeopardize steady state power operation (the design of the system is such that the power source must be removed from service to conduct the surveillance). The 24 hours is intended to indicate an outage of sufficient duration to allow scheduling and proper performance of the surveillance.

The above requirements of the surveillance frequency are based on guidance provided in Generic Letter 91-09.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 228
License No. DPR-62

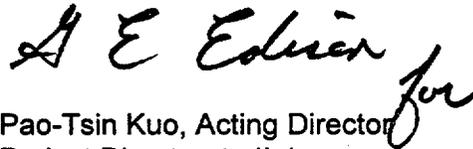
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated November 15, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 228, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "A E Edman for". The signature is written in a cursive style.

Pao-Tsin Kuo, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 29, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 228

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

XII

3/4 8-15

Insert Pages

XII

3/4 8-15

B 3/4 8-2

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.7 PLANT SYSTEMS (Continued)</u>	
3/4.7.3 FLOOD PROTECTION.....	B 3/4 7-1h
3/4.7.4 REACTOR CORE ISOLATION COOLING SYSTEM.....	B 3/4 7-1h
3/4.7.5 SNUBBERS.....	B 3/4 7-2
3/4.7.6 SEALED SOURCE CONTAMINATION.....	B 3/4 7-4
3/4.7.7 FIRE SUPPRESSION SYSTEMS (Deleted).....	B 3/4 7-4
3/4.7.8 FIRE BARRIER PENETRATIONS (Deleted).....	B 3/4 7-5
<u>3/4.8 ELECTRICAL POWER SYSTEMS.....</u>	B 3/4 8-1
3/4.8.2.5 REACTOR PROTECTION SYSTEM ELECTRIC POWER MONITORING....	B 3/4 8-2
<u>3/4.9 REFUELING OPERATIONS</u>	
3/4.9.1 REACTOR MODE SWITCH.....	B 3/4 9-1
3/4.9.2 INSTRUMENTATION.....	B 3/4 9-1
3/4.9.3 CONTROL ROD POSITION.....	B 3/4 9-1
3/4.9.4 DECAY TIME.....	B 3/4 9-1
3/4.9.5 COMMUNICATIONS.....	B 3/4 9-1
3/4.9.6 CRANE AND HOIST OPERABILITY.....	B 3/4 9-2
3/4.9.7 CRANE TRAVEL-SPENT FUEL STORAGE POOL.....	B 3/4 9-2
3/4.9.8 WATER LEVEL-REACTOR VESSEL, and	
3/4.9.9 WATER LEVEL-REACTOR FUEL STORAGE POOL.....	B 3/4 9-2
3/4.9.10 CONTROL ROD REMOVAL.....	B 3/4 9-2
<u>3/4.10 SPECIAL TEST EXCEPTIONS</u>	
3/4.10.1 PRIMARY CONTAINMENT INTEGRITY.....	B 3/4 10-1
3/4.10.2 ROD SEQUENCE CONTROL SYSTEM (DELETED).....	B 3/4 10-1
3/4.10.3 SHUTDOWN MARGIN DEMONSTRATIONS.....	B 3/4 10-1
3/4.10.4 RECIRCULATION LOOPS.....	B 3/4 10-1
3/4.10.5 PLANT SERVICE WATER.....	B 3/4 10-1

ELECTRICAL POWER SYSTEMS

REACTOR PROTECTION SYSTEM ELECTRIC POWER MONITORING

LIMITING CONDITION FOR OPERATION

3.8.2.5 Two RPS electric power monitoring channels for each inservice RPS MG set or alternate source shall be OPERABLE.

APPLICABILITY: Whenever the respective power supply is supplying power to a RPS bus.

ACTION:

- a. With one RPS electric power monitoring channel for an inservice RPS MG set or alternate power supply inoperable, restore the inoperable channel to OPERABLE status within 72 hours or remove the associated RPS MG set or alternate power supply from service.
- b. With both RPS electric power monitoring channels for an inservice RPS MG set or alternate power supply inoperable, restore at least one to OPERABLE status within 30 minutes or remove the associated RPS MG set or alternate power supply from service.

SURVEILLANCE REQUIREMENTS

4.8.2.5 The above specified RPS power monitoring system instrumentation shall be determined OPERABLE:

- a. By performance of a CHANNEL FUNCTIONAL TEST each time the plant is in COLD SHUTDOWN for a period of more than 24 hours, unless performed in the previous 6 months, and
- b. At least once per 18 months by demonstrating the OPERABILITY of over-voltage, under-voltage, and under-frequency protective instrumentation by performance of a CHANNEL CALIBRATION including simulated automatic actuation of the protective relays, tripping logic, and output circuit breakers and verifying the following setpoints:

	<u>RPS MG SET</u>	<u>ALTERNATE SOURCE</u>
1. Over-voltage	≤ 129 VAC	≤ 132 VAC
2. Under-voltage	≥ 105 VAC	≥ 108 VAC
3. Under-frequency	≥ 57 Hz	≥ 57 Hz

3/4.8 ELECTRICAL POWER SYSTEMS

BASES

3/4.8.2.5 REACTOR PROTECTION SYSTEM ELECTRIC POWER MONITORING

The CHANNEL FUNCTIONAL TEST is only required to be performed while the plant is in a condition in which the loss of the RPS bus will not jeopardize steady state power operation (the design of the system is such that the power source must be removed from service to conduct the surveillance). The 24 hours is intended to indicate an outage of sufficient duration to allow scheduling and proper performance of the surveillance.

The above requirements of the surveillance frequency are based on guidance provided in Generic Letter 91-09.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 198 TO FACILITY OPERATING LICENSE NO. DPR-71
AND AMENDMENT NO. 228 TO FACILITY OPERATING LICENSE NO. DPR-62
CAROLINA POWER & LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated November 15, 1995, Carolina Power and Light (CP&L) proposed a revision to the Technical Specifications (TS) for Brunswick Steam Electric Plant (BSEP), Unit Nos. 1 and 2. The revision pertains to modification of the channel functional test interval for the Electrical Protective Assemblies (EPAs) in the Reactor Protection System (RPS). This proposed change is a TS line item improvement per guidance in Generic Letter (GL) 91-09, "Modification of Surveillance Interval for the Electrical Protective Assemblies in Power Supplies for the Reactor Protection System." The proposed TS change includes TS 4.8.2.5 and associated TS Bases.

When performing an EPA surveillance test at power operation, a half scram is expected by design. The half scram is obtained from power interruption while performing a "dead bus transfer" of the RPS bus to the alternate power supply. In the past, licensees have experienced problems resetting the half scram, resulting in inadvertent scrams and group isolations. This result challenges safety systems and eventually could cause system degradation. By the proposed change, EPA testing will be performed with the unit in cold shutdown, thereby reducing the possibility of inadvertent trips and challenges to safety systems. Also, the licensee reported by phone on October 14, 1997, that the logic cards were upgraded during February 1997 with third generation logic cards with low set point drift, which can support the change in surveillance interval from every six months to every cold shut down.

2.0 EVALUATION

Redundant EPAs are provided to monitor the output of each RPS motor-generator (MG) set power source. Breakers located between the MG sets and the RPS are tripped upon sensing an abnormal MG set output voltage or frequency. This protects the RPS from the effects of continuous operation with a degraded power source. Because the RPS is not qualified for operation under degraded power source conditions, the EPAs preclude the potential for RPS system failures or failure of other safety actions due to abnormal MG set electrical output conditions.

The EPA channel functional test requires the licensee to transfer power for the RPS from the associated MG set to the alternate power supply. Because the transfer of RPS power involves a "dead-bus transfer," power is momentarily interrupted, which causes a half scram or group isolations. As an alternative, the licensee could perform tests without a bus transfer, but this

procedure also results in a momentary interruption of power to the RPS when each EPA channel is tripped during the channel functional test. At many boiling-water reactor (BWR) plants, licensees have encountered problems with the reset of the half-trip resulting in inadvertent scrams and group isolations that challenge safety systems.

The Niagra Mohawk Power Corporation provided an analysis for the Nine Mile Point Plant in a proposal submitted on December 15, 1988, that calculated the safety risks and benefits of the TS change to the RPS EPA surveillance interval. Also during the review of a similar TS change for Dresden Nuclear Power Station, the staff concluded that this TS change applies generically for BWRs as line item TS improvement. This conclusion formed the basis for the staff's issuance of Generic Letter (GL) 91-09 to provide the guidance for increasing the surveillance interval for EPA testing. Based on this guidance, the licensee proposed that the test interval for the EPAs be changed from every 6 months to each time the plant is in cold shutdown for a period of more than 24 hours unless performed in the previous 6 months. Also, the latest revision of the BWR standard Technical Specifications (STS) requires EPA testing in accordance with the guidance in GL 91-09. Therefore, the staff finds the licensee's proposed change to TS 4.8.2.5 acceptable as it is consistent with the previous staff approval.

The licensee also proposed corresponding changes to the TS Bases that would reflect the change to TS 4.8.2.5. The staff finds these changes acceptable.

Based upon a review of the licensee's submittal, the staff concludes that the proposed change to the BSEP Units 1 and 2 TS for EPA surveillance testing is consistent with the BWR STS and GL 91-09 guidance and as such will reduce the likelihood of inadvertent trips and challenges to safety systems. Therefore, the change to the TS and associated Bases is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 34887). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Salicrup

Date: May 29, 1998