



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

May 21, 1993

Docket Nos. 50-325  
and 50-324

Mr. R. A. Anderson  
Vice President  
Brunswick Steam Electric Plant  
Post Office Box 10429  
Southport, North Carolina 28461

Dear Mr. Anderson:

SUBJECT: ISSUANCE OF AMENDMENT NO. 162 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE NO. DPR-62 REGARDING NEUTRON MONITORING SURVEILLANCE REQUIREMENTS BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2, (TAC NOS. M85136 AND M85137)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 162 to Facility Operating License No. DPR-71 and Amendment No. 193 to Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant, Units 1 and 2. The amendments change the Technical Specifications in response to your submittal dated December 8, 1992.

The amendments change the Technical Specifications (TS) to revise the neutron monitoring instrumentation surveillance requirements associated with existing footnote (d) to TS Tables 4.3.1-1 and 4.3.4-1 to clarify that, when changing from Operational Condition 1 to Operational Condition 2, the performance of the required surveillance within 12 hours is not required if it was performed within the previous 7 days.

In addition, a new footnote (i), replacing footnote (d) on the average power range monitor (APRM) upscale (fixed) trip functional test frequency, would be incorporated into TS Table 4.3.4-1 to clarify that, when changing from Operational Condition 1 to Operational Condition 2, the performance of the required surveillance within 12 hours is not required if it was performed within the previous 92 days.

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May 21, 1993

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A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register Notice.

Sincerely,

ORIGINAL SIGNED BY:

Patrick D. Milano, Senior Project Manager  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 162 to License No. DPR-71
2. Amendment No. 193 to License No. DPR-62
3. Safety Evaluation

cc w/enclosures:  
See next page

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DATE	05/12/93	05/12/93	05/12/93	05/21/93	05/24/93

Document Name: BRN85136.AMD

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Brunswick Steam Electric Plant  
Units 1 and 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 162  
License No. DPR-71

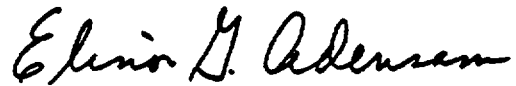
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated December 8, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 162, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Elinor G. Adensam, Director  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 21, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 162

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 3-9  
3/4 3-51  
3/4 3-52

Insert Pages

3/4 3-9  
3/4 3-51  
3/4 3-52

TABLE 4.3.4-1 (Continued)

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

NOTES

- (a) CHANNEL CALIBRATIONS are electronic.
- (b) This calibration shall consist of the adjustment of the APRM flow biased setpoint to conform to a calibrated flow signal.
- (c) Within 24 hours prior to startup, if not performed within the previous 7 days.
- (d) When changing from OPERATIONAL CONDITION 1 to OPERATIONAL CONDITION 2, perform the required surveillance within 12 hours after entering OPERATIONAL CONDITION 2, if not performed within the previous 7 days.
- (e) Placement of Reactor Mode Switch into Startup/Hot Standby position is permitted for the purpose of performing the required surveillance prior to withdrawal of control rods for the purpose of bringing the reactor to criticality.
- (f) Placement of Reactor Mode Switch into the Shutdown or Refuel position is permitted for the purpose of performing the required surveillance provided all control rods are fully inserted and the vessel head bolts are tensioned.
- (g) When THERMAL POWER is greater than the preset power level of the RWM and RSCS.
- (h) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.
- (i) When changing from OPERATIONAL CONDITION 1 to OPERATIONAL CONDITION 2, perform the required surveillance within 12 hours after entering OPERATIONAL CONDITION 2, if not performed within the previous 92 days.

TABLE 4.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

NOTES

- (a) Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (b) Within 24 hours prior to startup, if not performed within the previous 7 days.
- (c) The IRM channels shall be compared to the APRM channels and the SRM instruments for overlap during each startup, if not performed within the previous 7 days.
- (d) When changing from OPERATIONAL CONDITION 1 to OPERATIONAL CONDITION 2, perform the required surveillance within 12 hours after entering OPERATIONAL CONDITION 2, if not performed within the previous 7 days.
- (e) This calibration shall consist of the adjustment of the APRM readout to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER is greater than or equal to 25% of RATED THERMAL POWER.
- (f) This calibration shall consist of the adjustment of the APRM flow-biased setpoint to conform to a calibrated flow signal.
- (g) The LPRMs shall be calibrated at least once per effective full power month (EFPM) using the TIP system.
- (h) This calibration shall consist of a physical inspection and actuation of these position switches.
- (i) Instrument alignment using a standard current source.
- (j) Calibration using a standard radiation source.
- (k) The transmitter channel check is satisfied by the trip unit channel check. A separate transmitter check is not required.
- (l) Transmitters are exempted from the monthly channel calibration.
- (m) Placement of Reactor Mode Switch into the Startup/Hot Standby position is permitted for the purpose of performing the required surveillance prior to withdrawal of control rods for the purpose of bringing the reactor to criticality.
- (n) Placement of Reactor Mode Switch into the Shutdown or Refuel position is permitted for the purpose of performing the required surveillance provided all control rods are fully inserted and the vessel head bolts are tensioned.
- (o) Surveillance is not required when THERMAL POWER is less than 30% of RATED THERMAL POWER.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 193  
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated December 8, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 193, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Elinor G. Adensam*

Elinor G. Adensam, Director  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 21, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 193

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 3-9  
3/4 3-51  
3/4 3-52

Insert Pages

3/4 3-9  
3/4 3-51  
3/4 3-52

TABLE 4.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

NOTES

- (a) Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (b) Within 24 hours prior to startup, if not performed within the previous 7 days.
- (c) The IRM channels shall be compared to the APRM channels and the SRM instruments for overlap during each startup, if not performed within the previous 7 days.
- (d) When changing from OPERATIONAL CONDITION 1 to OPERATIONAL CONDITION 2, perform the required surveillance within 12 hours after entering OPERATIONAL CONDITION 2, if not performed within the previous 7 days.
- (e) This calibration shall consist of the adjustment of the APRM readout to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER greater than or equal to 25% of RATED THERMAL POWER.
- (f) This calibration shall consist of the adjustment of the APRM flow-biased setpoint to conform to a calibrated flow signal.
- (g) The LPRMs shall be calibrated at least once per effective full power month (EFPM) using the TIP system.
- (h) This calibration shall consist of a physical inspection and actuation of these position switches.
- (i) Instrument alignment using a standard current source.
- (j) Calibration using a standard radiation source.
- (k) The transmitter channel check is satisfied by the trip unit channel check. A separate transmitter check is not required.
- (l) Transmitters are exempted from the monthly channel calibration.
- (m) Placement of Reactor Mode Switch into the Startup/Hot Standby position is permitted for the purpose of performing the required surveillance prior to withdrawal of control rods for the purpose of bringing the reactor to criticality.
- (n) Placement of Reactor Mode Switch into the Shutdown or Refuel position is permitted for the purpose of performing the required surveillance provided all control rods are fully inserted and the vessel head bolts are tensioned.
- (o) Surveillance is not required when THERMAL POWER is less than 30% of RATED THERMAL POWER.

TABLE 4.3.4-1

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
1. <u>APRM</u>				
a. Upscale (Flow Biased)	NA	S/U (c)	R (b) (a)	1
b. Inoperative	NA	S/U (c) (e) M, Q (f)	NA	1, 2, 5
c. Downscale	NA	S/U (c)	NA	1
d. Upscale (Fixed)	NA	S/U (c) (e) M, Q (i) (f)	R (a)	2, 5
2. <u>ROD BLOCK MONITOR</u>				
a. Upscale	NA	S/U (c) M	R (a)	1 (g)
b. Inoperative	NA	S/U (c) Q	NA	1 (g)
c. Downscale	NA	S/U (c) M	R (a)	1 (g)
3. <u>SOURCE RANGE MONITORS</u>				
a. Detector not full in	NA	S/U (c) W (d)	NA	2, 5
b. Upscale	NA	S/U (c) W (d)	NA	2, 5
c. Inoperative	NA	S/U (c) W (d)	NA	2, 5
d. Downscale	NA	S/U (c) W (d)	NA	2, 5
4. <u>INTERMEDIATE RANGE MONITORS</u>				
a. Detector not full in	NA	S/U (c) (e) W (d)	NA	2
	NA	W (f)	NA	5
b. Upscale	NA	S/U (c) W (d)	NA	2
	NA	W	NA	5
c. Inoperative	NA	S/U (c) W (d)	NA	2
	NA	W	NA	5
d. Downscale	NA	S/U (c) W (d)	NA	2
	NA	W	NA	5
5. <u>SCRAM DISCHARGE VOLUME</u>				
a. Water Level - High	NA	Q	R	1, 2, 5 (h)

TABLE 4.3.4-1 (Continued)

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

NOTES

- (a) CHANNEL CALIBRATIONS are electronic.
- (b) This calibration shall consist of the adjustment of the APRM flow biased setpoint to conform to a calibrated flow signal.
- (c) Within 24 hours prior to startup, if not performed within the previous 7 days.
- (d) When changing from OPERATIONAL CONDITION 1 to OPERATIONAL CONDITION 2, perform the required surveillance within 12 hour. after entering OPERATIONAL CONDITION 2, if not performed within the previous 7 days.
- (e) Placement of Reactor Mode Switch into Startup/Hot Standby position is permitted for the purpose of performing the required surveillance prior to withdrawal of control rods for the purpose of bringing the reactor to criticality.
- (f) Placement of Reactor Mode Switch into the Shutdown or Refuel position is permitted for the purpose of performing the required surveillance provided all control rods are fully inserted and the vessel head bolts are tensioned.
- (g) When THERMAL POWER is greater than the preset power level of the RWM and RSCS.
- (h) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.
- (i) When changing from OPERATIONAL CONDITION 1 to OPERATIONAL CONDITION 2, perform the required surveillance within 12 hours after entering OPERATIONAL CONDITION 2, if not performed within the previous 92 days.

TABLE 4.3.4-1

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
1. <u>APRM</u>				
a. Upscale (Flow Biased)	NA	S/U <sup>(c)</sup>	R <sup>(b)(a)</sup>	1
b. Inoperative	NA	S/U <sup>(c)(e)</sup> <sup>M</sup> , Q <sup>(f)</sup>	NA	1, 2, 5
c. Downscale	NA	S/U <sup>(c)</sup>	NA	1
d. Upscale (Fixed)	NA	S/U <sup>(c)(e)</sup> <sup>M</sup> , Q <sup>(i)(f)</sup>	R <sup>(a)</sup>	2, 5
2. <u>ROD BLOCK MONITOR</u>				
a. Upscale	NA	S/U <sup>(c)</sup> , M	R <sup>(a)</sup>	1 <sup>(g)</sup>
b. Inoperative	NA	S/U <sup>(c)</sup> , Q	NA	1 <sup>(g)</sup>
c. Downscale	NA	S/U <sup>(c)</sup> , M	R <sup>(a)</sup>	1 <sup>(g)</sup>
3. <u>SOURCE RANGE MONITORS</u>				
a. Detector not full in	NA	S/U <sup>(c)</sup> , W <sup>(d)</sup>	NA	2, 5
b. Upscale	NA	S/U <sup>(c)</sup> , W <sup>(d)</sup>	NA	2, 5
c. Inoperative	NA	S/U <sup>(c)</sup> , W <sup>(d)</sup>	NA	2, 5
d. Downscale	NA	S/U <sup>(c)</sup> , W <sup>(d)</sup>	NA	2, 5
4. <u>INTERMEDIATE RANGE MONITORS</u>				
a. Detector not full in	NA	S/U <sup>(c)(e)</sup> , W <sup>(d)</sup>	NA	2
	NA	W <sup>(f)</sup>	NA	5
b. Upscale	NA	S/U <sup>(c)</sup> , W <sup>(d)</sup>	NA	2
	NA	W	NA	5
c. Inoperative	NA	S/U <sup>(c)</sup> , W <sup>(d)</sup>	NA	2
	NA	W	NA	5
d. Downscale	NA	S/U <sup>(c)</sup> , W <sup>(d)</sup>	NA	2
	NA	W	NA	5
5. <u>SCRAM DISCHARGE VOLUME</u>				
a. Water Level - High	NA	Q	R	1, 2, 5 <sup>(h)</sup>



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 162 TO FACILITY OPERATING LICENSE NO. DRP-71  
AND AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE NO. DPR-62  
CAROLINA POWER & LIGHT COMPANY  
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated December 8, 1992, the Carolina Power & Light Company (the licensee) submitted a request for changes to the Technical Specifications (TS) for Brunswick Steam Electric Plant, Units 1 and 2 (BSEP).

The current BSEP TS requirements for the neutron monitoring system are provided in two sections: (1) the reactor protection system (RPS) instrumentation surveillance requirements contained in TS Table 4.3.1-1, "Reactor Protection System Instrumentation Surveillance Requirements," and (2) the control rod withdrawal block (CRWB) instrumentation surveillance requirements contained in TS Table 4.3.4-1, "Control Rod Withdrawal Block Instrumentation Surveillance Requirements."

The requested amendments change the TS to revise the neutron monitoring instrumentation surveillance requirements associated with existing footnote (d) to TS Tables 4.3.1-1 and 4.3.4-1 to clarify that, when changing from Operational Condition 1 to Operational Condition 2, the performance of the required surveillance within 12 hours is not required if it was performed within the previous surveillance interval of 7 days as specified for those instruments in Operational Condition 2.

In addition, a new footnote (i), replacing footnote (d) on the average power range monitor (APRM) upscale (fixed) trip functional test frequency, would be incorporated into TS Table 4.3.4-1 to clarify that, when changing from Operational Condition 1 to Operational Condition 2, the performance of the required surveillance within 12 hours is not required if it was performed within the previous surveillance interval of 92 days as specified for those instruments in Operational Condition 2.



## 2.0 EVALUATION

The neutron monitoring system at BSEP provides for continuous monitoring of the neutron flux levels in the reactor at all power levels and consists of three subsystems: the APRMs, intermediate range monitors (IRMs), and source range monitors (SRMs). The system provides indication on the local and control room panels and generates trips whenever the monitored flux levels reach the predetermined trip settings. The trip functions associated with the proposed TS revisions are:

- (1) the IRM neutron flux high and inoperative scram inputs to the RPS;
- (2) the APRM neutron flux high 15 percent trip input to the RPS;
- (3) the APRM neutron flux upscale (fixed) trip input to the CRWB system; and,
- (4) the IRM and SRM detector not full in, upscale trip, inoperative trip, and downscale trip inputs to the CRWB system.

### 2.1 Existing Surveillance Requirements:

A weekly (7-day) channel functional test is required in Operational Condition 2 for the IRMs and APRMs associated with the RPS and for the SRMs and IRMs associated with the CRWB system. A channel functional test for APRMs associated with the neutron flux upscale (fixed) trip input to the CRWB system is required on a quarterly (92 days) basis.

Footnote (d) to TS Tables 4.3.1-1 and 4.3.4-1 currently requires that a surveillance (channel functional test) be performed within 12 hours after changing the operational mode of the plant from Operational Condition 1 to Operational Condition 2.

### 2.2 Proposed TS Modifications:

Footnote (d) to Tables 4.3.1-1 and 4.3.4-1 will be revised to read:

When changing from Operational Condition 1 to Operational Condition 2 perform the required surveillance within 12 hours after entering Operational Condition 2, if not performed within the previous 7 days.

Therefore, the revised footnote will make clear that, when changing from Operational Condition 1 to Operational Condition 2, it is not necessary to perform the required surveillance test if such a test was performed within the last 7 days, which is the scheduled surveillance interval specified for these instruments in Operational Condition 2.

For APRM upscale (fixed) trip input to the CRWB system, the scheduled interval for Operational Condition 2 is 92 days; therefore, a new footnote (i) is proposed to be added to Table 4.3.4-1. This footnote will read:

When changing from Operational Condition 1 to Operational Condition 2, perform the required surveillance within 12 hours after entering Operational Condition 2, if not performed within the previous 92 days.

Therefore, the new footnote will make clear that, when changing from Operational Condition 1 to Operational Condition 2, it is not necessary to perform the required surveillance test if such a test was performed within the last 92 days, which is a scheduled surveillance interval specified for these instruments in Operational Condition 2.

The scheduled surveillance test intervals of 7 days and 92 days, as described above for Operational Condition 2, were approved by the NRC in the past for the affected neutron monitoring instrumentation. If, within this interval, the plant condition is changed from Operational Condition 1 to Operational Condition 2, the staff believes that it is not necessary to repeat the surveillance tests if the tests were performed within the last 7 or 92 (as applicable) days because these tests would still be valid for the operating condition. The staff agrees with the licensee that by avoiding unnecessary tests, operator burden during startup could be reduced and additional personnel occupational radiation exposure could be eliminated.

The staff has reviewed the licensee's evaluation of this proposed TS modification and concludes that it is consistent with current TS requirements and is, therefore, acceptable.

Based on the above evaluation, the staff finds the proposed TS modification revising the surveillance requirements for neutron monitoring instruments acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes the Surveillance Requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a

proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 16217). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. V. Athavale

Date: May 21, 1993