Docket Nos. 50-325 and 50-324 DISTRIBUTION See attached list

Mr. Lynn W. Eury Executive Vice President Power Supply Carolina Power & Light Company Post Office Box 1551 Raleigh, North Carolina 27602

Dear Mr. Eury:

SUBJECT: ISSUANCE OF AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 181 TO FACILITY OPERATING LICENSE NO. DPR-62 REGARDING REMOVAL OF RESIDUAL HEAT REMOVAL HEAD SPRAY FLOW TRANSMITTER - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2, (TAC NOS. 76862 AND 76863)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 151 to Facility Operating License No. DPR-71 and Amendment No. 181 to Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal dated May 22, 1990, as supplemented by your submittal dated December 27, 1990.

The amendments change the Technical Specifications to delete Item 7, Residual Heat Removal Head Spray Flow, from Technical Specification Tables 3.3.5.2-1 and 4.3.5.2-1, dealing with Remote Shutdown Monitoring Instrumentation. The head spray mode of RHR is no longer in use at Brunswick and, as such, performing the required surveillances constitutes unnecessary personnel radiation exposure.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register Notice.

Sincerely,

Ronnie Lo/for

Ngoc B. Le, Project Manager Project Directorate II-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 151 to License No. DPR-71

2. Amendment No. 181 to License No. DPR-62

3. Safety Evaluation cc w/enclosures:

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NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

January 9, 1991

Docket Nos. 50-325 and 50-324

Mr. Lynn W. Eury
Executive Vice President
Power Supply
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Eury:

SUBJECT:

ISSUANCE OF AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-71 AND AMENDMENT NO. 181 TO FACILITY OPERATING LICENSE NO. DPR-62 REGARDING REMOVAL OF RESIDUAL HEAT REMOVAL HEAD SPRAY FLOW TRANSMITTER - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2, (TAC NOS. 76862 AND 76863)

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The amendments change the Technical Specifications to delete Item 7, Residual Heat Removal Head Spray Flow, from Technical Specification Tables 3.3.5.2-1 and 4.3.5.2-1, dealing with Remote Shutdown Monitoring Instrumentation. The head spray mode of RHR is no longer in use at Brunswick and, as such, performing the required surveillances constitutes unnecessary personnel radiation exposure.

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Sincerely,

Ngoc B. Le, Project Manager Project Directorate II-1

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 151 to License No. DPR-71

2. Amendment No. 181 to License No. DPR-62

3. Safety Evaluation

cc w/enclosures: See next page Mr. L. W. Eury Carolina Power & Light Company

cc:

Mr. Russell B. Starkey, Jr. Vice President Brunswick Nuclear Project P. O. Box 10429 Southport, North Carolina 28461

Mr. R. E. Jones, General Counsel Carolina Power & Light Company P. O. Box 1551 Raleigh, North Carolina 27602

Ms. Frankie Rabon Board of Commissioners P. O. Box 249 Bolivia, North Carolina 28422

Resident Inspector
U. S. Nuclear Regulatory Commission
Star Route 1
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Southport, North Carolina 28461

Regional Administrator, Region II U. S. Nuclear Regulatory Commission 101 Marietta Street, Suite 2900 Atlanta, Georgia 30323

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P. O. Box 27687
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Mr. J. L. Harness Plant General Manager Brunswick Steam Electric Plant P. O. Box 10429 Southport, North Carolina 28461 Brunswick Steam Electric Plant Units 1 and 2

Mr. H. A. Cole Special Deputy Attorney General State of North Carolina P. O. Box 629 Raleigh, North Carolina 27602

Mr. Robert P. Gruber
Executive Director
Public Staff - NCUC
P. O. Box 29520
Raleigh, North Carolina 27626-0520

AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-71 - BRUNSWICK, UNIT 1 AMENDMENT NO. 181 TO FACILITY OPERATING LICENSE NO. DPR-62 - BRUNSWICK, UNIT 2

Docket File NRC PDR Local PDR PDII-1 Reading S. Varga (14E4) G. Lainas E. Adensam P. Anderson N. Le D. Spaulding OGC D. Hagan (MNBB 3302)
G. Hill (4) (P1-137)
Wanda Jones (P-130A) J. Calvo (11D3)
R. Jones (8E23)
G. Thomas (8E23) ACRS (10) GPA/PA OC/LFMB Brunswick File

cc: Brunswick Service List



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20655

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 151 License No. DPR-71

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated May 22, 1990, and supplemented dated December 27, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 151 , are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Ronnie Lo/for

Elinor G. Adensam, Director Project Directorate II-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: January 9, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 151

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages	Insert Pages	
3/4 3-57	3/4 3-57	
3/4 3-58	3/4 3-58	

TABLE 3.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

FUNCTIONAL UNIT	READOUT LOCATION	MINIMUM CHANNELS OPERABLE
1. Reactor Vessel Pressure	RSP(a)	1
2. Reactor Vessel Water Level	RSP(a)	1
3. Suppression Chamber Water Level	RSP(a)	1
4. Suppression Chamber Water Temperature	RSP(a)	1
5. Drywell Pressure	RSP ^(a)	1
6. Drywell Temperature	RSP(a)	1
7. Residual Heat Removal System Flow	RSP(a)	1

⁽a) Remote Shutdown Panel, Reactor Building 20' Elevation

TABLE 4.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION
1. Reactor Vessel Pressure	M	Q
2. Reactor Vessel Water Level NO17 Instrument Loop: Remaining Instruments:	NA M	Q Q
3. Suppression Chamber Water Level	М	R
4. Suppression Chamber Water Temperature	М	R
5. Drywell Pressure	М	Q
6. Drywell Temperature	М	R
7. Residual Heat Removal System Flow	М	Q



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 181 License No. DPR-62

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Carolina Power & Light Company (the licensee), dated May 22, 1990, and supplement dated December 27, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 181, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Ronnie Lo/for

Elinor G. Adensam, Director Project Directorate II-1 Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: January 9, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 181

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages	<u>Insert Pages</u>
3/4 3-57	3/4 3-57
3/4 3-58	3/4 3-58

TABLE 3.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

FUNCTIONAL UNIT	READOUT LOCATION	MINIMUM CHANNELS OPERABLE
1. Reactor Vessel Pressure	RSP(a)	1
2. Reactor Vessel Water Level	RSP(a)	1
3. Suppression Chamber Water Level	RSP(a)	1
4. Suppression Chamber Water Temperature	RSP(a)	1
5. Drywell Pressure	RSP(a)	1
6. Drywell Temperature	RSP(a)	1
7. Residual Heat Removal System Flow	RSP(a)	1

⁽a) Remote Shutdown Panel, Reactor Building 20' Elevation

TABLE 4.3.5.2-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUN	CTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION
1.	Reactor Vessel Pressure	M	Q
2.	Reactor Vessel Water Level NO17 Instrument Loop: Remaining Instruments:	NA M	Q Q
3.	Suppression Chamber Water Level	М	R
4.	Suppression Chamber Water Temperature	М	R
5.	Drywell Pressure	М	Q
6.	Drywell Temperature	М	R
7.	Residual Heat Removal System Flow	М	Q



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-71

AND AMENDMENT NO. 181 TO FACILITY OPERATING LICENSE NO. DPR-62

CAROLINA POWER & LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2

DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated May 22, 1990, Carolina Power & Light Company (the licensee) submitted a request for an amendment to the Brunswick Steam Electric Plant, Units 1 and 2 (Brunswick), Technical Specifications (TS). Additional information was also provided by a letter dated December 27, 1990, which provided TS pages that did not change the proposed no sigificant hazards consideration publiched in the Federal Register

The proposed amendent deletes Item 7, residual heat removal (RHR) head spray flow, from TS Tables 3.3.5.2-1 and 4.3.5.2-1, which deals with remote shutdown monitoring instrumentation. The head spray mode of RHR is no longer in use at Brunswick.

2.0 EVALUATION

During reactor shutdown, the purpose of the head spray mode of the RHR is to spray water in the reactor vessel head area, thus providing a rapid vessel head cooldown. The head spray mode of RHR is a low flow, low pressure system designed to supply 615 gpm of water to the vessel steam dome, through the head spray nozzle, at less than 129 psig. Brunswick was designed with this capability when it was anticipated that reactor vessel head conditions would be the critical path for beginning a refueling outage. There are no safety-related functions associated with the head spray mode of the RHR, nor is use of this capability addressed in the Emergency Operating Procedures. The head spray mode of the RHR is not used for accident mitigation, normal operation, or shutdown at Brunswick. In addition, its use for shutting down the unit was optional.

In Unit 2, the head spray piping to the reactor head is already removed. By FSAR Amendment No. 7, dated June 1, 1989, the licensee deactivated the head spray mode of the RHR for both units. Even though the head spray mode was deactivated, the TS required periodic surveillance for the flow transmitter FT 3339 on both units. We have reviewed the information submitted for deletion of flow transmitter FT 3339 of the Brunswick, Units 1 and 2. Based on this review, we conclude that appropriate documentation was submitted to justify the proposed TS changes.

Table 3.3.5.2-1 of the TS lists the remote shutdown instrumentation required to be operable when the unit is in Operational Conditions 1, 2, or 3. Table 4.3.5.2-1 provides the surveillance requirements associated with this instrumentation. Currently, Item 7 of these tables requires the RHR heat removal head spray flow instrumentation FT 3339 to be operable and establishes monthly and quarterly surveillance requirements to demonstrate this operability. As stated above, the head spray mode of the RHR is no longer in use at Brunswick and has been deactivated on both units.

Therefore, operability of the RHR head spray flow instrumentation is not required and performing the required surveillance constitutes unnecessary personnel radiation exposure. Hence the licensee's proposal to delete Item 7 of the RHR head spray flow from Tables 3.3.5.2-1 and 4.3.5.2-1 dealing with remote and shutdown instrumentation is acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

These amendments change a requirement with respect to installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20 and change the surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released off site and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that these amendments involve no significant hazards consideration which was published in the <u>Federal Register</u> (55 FR 30292) on July 25, 1990, and consulted with the <u>State of North Carolina</u>. No public comments or requests for hearing were received, and the State of North Carolina did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 9, 1991

Principal Contributor: G. Thomas