

## **Constellation Nuclear**

**Nine Mile Point  
Nuclear Station**

*A Member of the  
Constellation Energy Group*

December 21, 2001  
NMP1L 1634

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

RE: Nine Mile Point Unit 1  
Docket No. 50-220  
DPR-63

***Subject: Special Report, Channel 12 Containment Hydrogen Monitoring System  
Inoperable Due to Inoperable Check Valve***

Gentlemen:

In accordance with Action Statement 4.a of Nine Mile Point Unit 1 Technical Specification Table 3.6.11-2, "Accident Monitoring Instrumentation Action Statements," Nine Mile Point Nuclear Station (NMPNS) is submitting the following Special Report concerning the inoperability of Channel #12 of the Containment Hydrogen Monitoring System.

### **Description of Event**

On December 13, 2001, at approximately 1520 hours, Check Valve 201.2-70 failed reverse flow testing. The check valve was declared inoperable and Manual Blocking Valve 201.2-601 was closed. Check Valve 201.2-70 is in the Torus return line from the Channel #12 Containment Hydrogen Monitoring System. With the manual blocking valve closed, Channel #12 was declared inoperable and Action Statement 4.a of Technical Specification Table 3.6.11-2 was entered. On December 14, 2001, a local leak rate test was successfully completed on Check Valve 201.2-70, indicating that the check valve was indeed shut. The check valve was declared operable and Manual Blocking Valve 201.2-601 was opened. At 1725 hours Channel #12 was returned to operable. Channel #11 of the Containment Hydrogen Monitoring System remained operable during the period that Channel #12 was inoperable.

### **Cause of Event**

Channel #12 of the Containment Hydrogen Monitoring System was declared inoperable when the manual blocking valve was closed due to Check Valve 201.2-70 failing reverse flow testing. An evaluation of the failed reverse flow test concluded that the cause of the failure was inadequate test methodology.

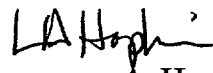
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**Corrective Actions**

1. A local leak rate check of valve 201.2-70 was performed successfully.
2. Revised reverse flow testing methodology for Check Valve 201.2-70 will be incorporated into the testing procedure N1-ST-Q18, "H2 -O2 Monitoring System Valves and Associated Reactor Building Closed Loop Cooling System Valve Operability Test," by February 20, 2002.

Very truly yours,

  
Lawrence A. Hopkins  
Unit 1 Plant General Manager

LAH/KLE/cld

cc: Mr. H. J. Miller, NRC Regional Administrator, Region I  
Mr. G. K. Hunegs, NRC Senior Resident Inspector  
Records Management