

March 4, 2002

Mr. R. T. Ridenoure
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 - ISSUANCE OF AMENDMENT RE:
ADDITION OF TOPICAL REPORT REFERENCES TO TS 5.9.5, "CORE
OPERATING LIMITS REPORT" (TAC NO. MB3449)

Dear Mr. Ridenoure:

The Commission has issued the enclosed Amendment No. 203 to Facility Operating License No. DPR-40 for the Fort Calhoun Station, Unit No. 1. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated November 21, 2001.

The amendment adds three topical report references to TS 5.9.5, "Core Operating Limits Report."

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Alan B. Wang, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosures: 1. Amendment No. 203 to DPR-40
2. Safety Evaluation

cc w/encls: See next page

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OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 203
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Omaha Public Power District (the licensee) dated November 21, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 203, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Stephen Dembek, Chief, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: March 4, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 203

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

REMOVE

5-10

5-11

INSERT

5-10

5-11

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 203 TO FACILITY OPERATING LICENSE NO. DPR-40

OMAHA PUBLIC POWER DISTRICT
FORT CALHOUN STATION, UNIT NO. 1

DOCKET NO. 50-285

1.0 INTRODUCTION

By application dated November 21, 2001, Omaha Public Power District (OPPD) requested changes to the Technical Specifications (TSs, Appendix A to Facility Operating License No. DPR-40) for the Fort Calhoun Station, Unit No. 1. The requested changes would add three analytical methods to the list of approved core operating limits analytical methods in TS 5.9.5, "Core Operating Limits Report," for Fort Calhoun Station Unit 1. This proposed TS change will facilitate future reload calculations to support refueling outages and provides additional flexibility.

2.0 EVALUATION

Fort Calhoun Station is planning a refueling outage in May 2002. Forty Westinghouse operating fuel assemblies will be replaced with new Framatome ANP assemblies during the May 2002 refueling outage. This is in addition to the 53 Framatome ANP assemblies that were installed during the last refueling outage. Therefore, of the 133 fuel assemblies in the core, 93 of the assemblies will now be manufactured by Framatome. The remaining 40 Westinghouse fuel assemblies will be replaced during the Fall 2003 refueling outage. Framatome ANP has developed three analytical methodologies to perform reload analyses. The licensee has proposed to reference the following three methods in the TS:

- (1) EMF-2328(P)(A), "PWR Small Break LOCA Evaluation Model, S-RELAP5 Based," which was reviewed and approved by the staff for referencing by letter dated March 15, 2001 (Reference 1).
- (2) EMF-96-029(P)(A) Volume 1, EMF-96-029(P)(A) Volume 2, and EMF-96-029(P)(A) Attachment, "Reactor Analysis System for PWRs, Volume 1, Volume 2, and Attachment," which were reviewed and approved by the staff for referencing by letter dated October 29, 1996 (Reference 2).
- (3) EMF-2310(P)(A), "SRP Chapter 15 Non-LOCA Methodology for Pressurized Water Reactors," which was reviewed and approved by the staff for referencing by letter dated May 11, 2001 (Reference 3).

These three methodologies are applicable for the reload analyses at the Fort Calhoun Station and have been previously reviewed and approved by the staff. The use of these methodologies will enable replacement of defective fuel assemblies and result in an efficient core operation.

The proposed TS change will add these three topical reports to TS 5.9.5. These approved analytical methods ensure that core parameters remain within prescribed limitations defined by the plant's design basis and updated final safety analysis report accident analyses. Accordingly, the staff finds this proposal acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Nebraska State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

6.0 REFERENCES

1. Letter, NRC to Framatome ANP, Richland, Inc., "Acceptance for Referencing of Licensing Topical Report EMF-2328 (P)(A), Revision 0, 'PWR Small Break LOCA Evaluation Model, S-RELAP5 Based'," dated March 15, 2001.
2. Letter, NRC to Siemens Power Corporation, "Acceptance for Referencing of Topical Report EMF-96-029(P)(A), Vols. 1 and 2, 'Reactor Analysis System for PWRs'," dated October 29, 1996.

3. Letter, NRC to Framatome ANP, Richland, Inc., "Acceptance for Referencing of Licensing Topical Report EMF-2310(P)(A), Revision 0, 'SRP Chapter 15 Non-LOCA Methodology for Pressurized Water Reactors'," dated May 11, 2001.

Principal Contributor: K. Desai

Date: March 4, 2002

Ft. Calhoun Station, Unit 1

cc:

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