

APR 19 1988

Docket Nos. 50-325/324

Mr. E. E. Utley
Senior Executive Vice President
Power Supply and Engineering & Construction
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

SUBJECT: CORRECTION TO AMENDMENT NOS. 116 AND 144 - BRUNSWICK STEAM
ELECTRIC PLANT, UNITS 1 AND 2 (TAC NOS. 66048/66049)

On February 17, 1988, the Nuclear Regulatory Commission issued Amendment No. 116 to Facility Operating License No. DPR-71 and Amendment No. 144 to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. Inadvertently, that amendment was issued with incorrect amendment numbers on the bottom of two pages. The corrected pages are enclosed.

We regret any inconvenience this may have caused you.

Sincerely,

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Ernest D. Sylvester, Project Manager
Project Directorate II-1
Division of Reactor Projects I/II

Enclosure:
Corrected Amendment Pages

cc w/enclosures:
See next page

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DATE	: 4/14/88 : 4/19/88 : 4/19/88 :	:	:	:	:

Mr. E. E. Utley
Carolina Power & Light Company

Brunswick Steam Electric Plant
Units 1 and 2

cc:

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Raleigh, North Carolina 27603-2008

3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 PRIMARY CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR Part 100 during accident conditions.

3/4.6.1.2 PRIMARY CONTAINMENT LEAKAGE

The limitations on primary containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure of 49 psig, P_a . As an added conservatism, the measured overall integrated leakage rate is further limited to less than or equal to $0.75 L_a$ or $0.75 L_p$, as applicable, during performance of the periodic tests to account for possible degradation of the containment leakage barriers between leakage tests.

Operating experience with the main steam line isolation valves has indicated that degradation has occasionally occurred in the leak tightness of the valves; therefore, the special requirement for testing these valves.

Exemptions from the requirements of 10 CFR Part 50 have been granted for main steam isolation valve leak testing, testing of airlocks after each opening, and leakage calculation methods.

Appendix J, paragraph III.A.3 requires that all Type A (Containment Integrated Leak Rate) tests be performed in accordance with ANSI N45.4-1972, "Leakage Rate Testing of Containment Structures for Nuclear Reactors." ANSI N45.4-1972 requires that leakage calculations be performed using the Point-to-Point or Total Time method. ANSI N45.4-1972 has been revised to a new standard, ANSI/ANS 56.8-1981, "Containment System Leakage Testing," which incorporates the Mass-Point method for leakage calculations. Type A tests will be performed in conformance with ANSI N45.4-1972 but will use the Mass-Point method for calculation of leakage rates as described in ANSI/ANS 56.8-1981.

3/4.6.1.3 PRIMARY CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on PRIMARY CONTAINMENT INTEGRITY and leak rate given in Specifications 3.6.1.1 and 3.6.1.2. The specification makes allowances for the fact that there may be long periods of time when the air locks will be in a closed and secured position during reactor operation.

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