

February, 7, 2002

Mr. R. T. Ridenoure
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 - REQUEST FOR ADDITIONAL
INFORMATION (TAC NO. MB3654)

Dear Mr. Ridenoure:

The NRC staff has reviewed your amendment request dated December 14, 2001, and has determined that additional information is needed to complete our review. Enclosed is a request for additional information regarding the December 14, 2001, submittal. Please respond within 30 days of receipt of this letter. This request has been discussed with Richard Jaworski of your staff and he has agreed to this schedule. If you have any questions regarding this request, please contact me at (301) 415-1445.

Sincerely,
/RA/

Alan B. Wang, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosures: Request for Additional Information

cc w/encls: See next page

Ft. Calhoun Station, Unit 1

cc:

Winston & Strawn
ATTN: James R. Curtiss, Esq.
1400 L Street, N.W.
Washington, DC 20005-3502

Mr. Jack Jensen, Chairman
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Mr. Wayne Walker, Resident Inspector
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Fort Calhoun, NE 68023

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
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Arlington, TX 76011

Ms. Julia Schmitt, Section Administrator
Nebraska Health & Human Services
Systems
Division of Public Health Assurance
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Lincoln, Nebraska 68509-5007

Mr. Richard P. Clemens
Manager - Fort Calhoun Station
Omaha Public Power District
Fort Calhoun Station FC-1-1 Plant
Post Office Box 550
Fort Calhoun, NE 68023-0550

Mr. Mark T. Frans
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Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Fort Calhoun, NE 68023-0550

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REQUEST FOR ADDITIONAL INFORMATION

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT 1

DOCKET NO. 50-285

By letter dated December 14, 2001, Omaha Public Power District (OPPD/licensee) proposed to apply ASME Code Case N-640, to Appendix G to Section XI of the ASME Boiler and Pressure Vessel Code, as the underlying basis for the establishment of the pressure-temperature (P-T) limit curves. The proposed technical specification change seeks to employ a single P-T limit curve for the reactor pressure vessel heatup and cooldown. This single heatup/cooldown P-T limit curve is a composite curve generated from the most limiting pressure for a given temperature from a 100°F/hr cooldown curve (above 178°F), 50°F/hr cooldown curve, 75°F/hr heatup curve, and an isothermal curve. The NRC staff requires the following information to complete our review of your submittal.

For each curve (the 100°F/hr cooldown curve, the 50°F/hr cooldown curve, the 75°F/hr heatup curve, and the isothermal curve) used to develop the composite heatup/cooldown curve, provide the following data for each temperature point given in Table 1 of Westinghouse letter LTR-PS-01-26, Rev. 00:

K_{lm} - the crack tip stress intensity due to membrane (pressure) stresses for the 1/4 T and 3/4 T flaws;

K_{lt} - the crack tip stress intensity due to thermal stresses for the 1/4 T and 3/4 T flaws;

K_{lc} - the material fracture toughness at the crack tip of the 1/4 T and 3/4 T flaws;

$T_{1/4T}$ - the reactor pressure vessel (RPV) metal temperature at the tip of the 1/4 T flaw; and

$T_{3/4T}$ - the RPV metal temperature at the tip of the 3/4 T flaw.