

August 10, 1987

Dockets Nos. 50-325/324

DISTRIBUTION  
See next page

Mr. E. E. Utley  
Senior Executive Vice President  
Power Supply and Engineering & Construction  
Carolina Power & Light Company  
Post Office Box 1551  
Raleigh, North Carolina 27602

Dear Mr. Utley:

SUBJECT: ISSUANCE OF AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE  
NO. DPR-71 AND AMENDMENT NO. 137 TO FACILITY OPERATING LICENSE  
NO. DPR-62 - BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2,  
REGARDING PRIMARY CONTAINMENT ISOLATION VALVE TESTING (TAC  
NOS. 62750 AND 62751)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 110 to Facility Operating License No. DPR-71 and Amendment No. 137 to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal dated August 29, 1986, as supplemented July 16, 1987.

The amendments change the Technical Specifications (TS) to delete the requirement from Section 4.6.3.2 to test the primary containment isolation valves for operability only during the cold shutdown or refueling modes of operation. The primary containment isolation valves listed in TS Table 3.6.3-1 are capable of being tested during power operation. The amendments do not change the surveillance interval for these valves.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Ernest D. Sylvester, Project Manager  
Project Directorate II-1  
Division of Reactor Projects I/II

Enclosures:

1. Amendment No.110 to License No. DPR-71
2. Amendment No. 137 to License No. DPR-62
3. Safety Evaluation

cc w/enclosures:  
See next page  
OFFICIAL RECORD COPY

PD21:DRPR  
PAnderson  
7/ /87

PD21:DRPR *ES*  
ESylvester/sh  
7/29/87

*E*  
D: ~~PD21~~:DRPR  
EAdensam  
7/30/87

8708130093 870810  
PDR ADOCK 05000324  
P PDR

Mr. E. E. Utley Brunswick Steam Electric Plant  
Carolina Power & Light Company Units 1 and 2

cc:

Richard E. Jones, Esquire  
Carolina Power & Light Company  
336 Fayetteville Street  
Raleigh, North Carolina 27602

George F. Trowbridge, Esquire  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N. W.  
Washington, D. C. 20036

Mr. Charles R. Dietz  
Plant Manager  
Post Office Box 458  
Southport, North Carolina 28461

Mr. Franky Thomas, Chairman  
Board of Commissioners  
Post Office Box 249  
Bolivia, North Carolina 28422

Mrs. Chrys Baggett  
State Clearinghouse  
Budget and Management  
116 West Jones Street  
Raleigh, North Carolina 27603

Resident Inspector  
U. S. Nuclear Regulatory Commission  
Star Route 1  
Post Office Box 208  
Southport, North Carolina 28461

Regional Administrator, Region II  
U. S. Nuclear Regulatory Commission  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

Dayne H. Brown, Chief  
Radiation Protection Branch  
Division of Facility Services  
Department of Human Resources  
Post Office Box 12200  
Raleigh, North Carolina 27605



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110  
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power & Light Company (the licensee), dated August 29, 1986, as supplemented July 16, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

8708130100 870810  
PDR ADDCK 05000324  
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 110, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam, Director  
Project Directorate II-1  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 10, 1987

OFFICIAL RECORD COPY

*BCB for*  
PD21:DRPR  
PAnderson  
8/6/87

PD21:DRPR *EOS*  
ESylvester/sh  
7/30/87

OGC-B  
*Beach*  
8/4/87

*WJL* 8/4/87

*BCB*  
D:PD21:DRPR  
EAdensam  
8/6/87

ATTACHMENT TO LICENSE AMENDMENT NO. 110

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove Page

3/4 6-13

Insert Page

3/4 6-13

CONTAINMENT SYSTEMSSURVEILLANCE REQUIREMENTS

4.6.3.1 Each primary containment isolation valve specified in Table 3.6.3-1 shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair, or replacement work is performed on the valve or its associated actuator, control, or power circuit by performance of the cycling test and verification of isolation time.

4.6.3.2 Each isolation valve specified in Table 3.6.3-1 shall be demonstrated OPERABLE at least once per 18 months by verifying that on a containment isolation test signal each isolation valve actuates to its isolation position.

4.6.3.3 The isolation time of each power-operated or automatic valve specified in Table 3.6.3-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

4.6.3.4 Each reactor instrumentation system isolation valve shall be demonstrated OPERABLE at least once per 18 months by cycling each valve through at least one complete cycle of full travel.\*

---

\*The end of the current surveillance period for the surveillance requirements below may be extended beyond the time limit specified by Technical Specification 4.0.2a. After November 2, 1984, the plant shall not be operated in Operational Conditions 1, 2, or 3 until the surveillance requirements listed below have been completed. Upon accomplishment of the surveillances, the provisions of Technical Specification 4.0.2a shall apply.

Technical Specification 4.6.3.4; as applicable to excess flow check valves B21-F047C, B21-F047D, B21-F049C, and B21-F049D.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY  
DOCKET NO. 50-324  
BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 137  
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power & Light Company (the licensee), dated August 29, 1986, as supplemented July 16, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 137, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam, Director  
Project Directorate II-1  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 10, 1987

*Bob for*  
LA:PD21:DRPR  
PAnderson  
8/10/87

PM:PD21:DRPR  
ESlyvester *ES*  
7/31/87

OGC/B  
*[Signature]*  
8/4/87  
*[Signature]* 8/4/87

*Bob for*  
D:PD21:DRPR  
EAdensam  
8/10/87

ATTACHMENT TO LICENSE AMENDMENT NO. 137

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove Page

3/4 6-13

Insert Page

3/4 6-13

CONTAINMENT SYSTEMSSURVEILLANCE REQUIREMENTS

4.6.3.1 Each primary containment isolation valve specified in Table 3.6.3-1 shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair, or replacement work is performed on the valve or its associated actuator, control, or power circuit by performance of the cycling test and verification of isolation time.

4.6.3.2 Each isolation valve specified in Table 3.6.3-1 shall be demonstrated OPERABLE at least once per 18 months by verifying that on a containment isolation test signal each isolation valve actuates to its isolation position.

4.6.3.3 The isolation time of each power-operated or automatic valve specified in Table 3.6.3-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

4.6.3.4 Each reactor instrumentation system isolation valve shall be demonstrated OPERABLE at least once per 18 months by cycling each valve through at least one complete cycle of full travel.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE NO. DPR-71  
AND AMENDMENT NO. 137 TO FACILITY OPERATING LICENSE NO. DPR-62  
CAROLINA POWER & LIGHT COMPANY  
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-325 AND 50-324

1.0 INTRODUCTION

By letter dated August 29, 1986, as supplemented July 16, 1987, Carolina Power & Light Company (the licensee) submitted a request for changes to the Brunswick Steam Electric Plant, Units 1 and 2, Technical Specifications (TS) to permit operability testing of the primary containment isolation valves during any mode of reactor operation. The amendment would delete the words "during cold shutdown or refueling" from the surveillance requirements for containment isolation valves in TS Section 4.6.3.2.

The staff noted in the August 29, 1986 application that the licensee inadvertently left out a paragraph on the corrected TS page 3/4 6-13 for Brunswick Unit 1. By letter dated July 16, 1987, the licensee provided the corrected TS page to the staff. Therefore, the supplemental letter corrected an administrative error to the original application and does not change the initial no significant hazards consideration determination as published in the Federal Register.

2.0 EVALUATION

TS 4.6.3.2 requires that each primary containment isolation valve listed in TS Table 3.6.3-1 be demonstrated operable during "cold shutdown or refueling" at least once every 18 months. This operability test should verify that the valve actuates to the appropriate position upon receipt of a test signal. The licensee stated that this requirement limits the operational flexibility of the plant for those valves that are capable of being tested during power operation. The Brunswick Updated Final Safety Analysis Report, paragraph 7.3.1.1.9, states "The primary containment isolation and NSSS Shutoff System is testable during reactor operation. Isolation valves can be tested to assure that they are

8708130107 870810  
PDR ADDCK 05000324  
PDR  
P

capable of closing by operating manual switches in the Control Room and observing the position lights and any associated process effects." The proposed revision to the Brunswick TS, if approved, would delete the phrase "during cold shutdown and refueling" from Surveillance Requirement 4.6.3.2 and would read "Each isolation valve specified in Table 3.6.3-1 shall be demonstrated OPERABLE at least once per 18 months by verifying that on a containment isolation test signal each isolation valve actuates to its isolation position." The proposed TS amendment does not change the test interval and the staff, therefore, finds it acceptable. The proposed amendment makes Brunswick Units 1 and 2 TS consistent with the Standard TS for BWR-4s.

### 3.0 ENVIRONMENTAL CONSIDERATIONS

The amendments change the surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite; and that there should be no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 4.0 CONCLUSION

The Commission made a determination that these amendments involve no significant hazards consideration which was published in the FEDERAL REGISTER (51 FR 36084) on October 8, 1986, and consulted with the State of North Carolina. No public comments or requests for hearing were received, and the State of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Pulsipher

Dated: August 10, 1987

DISTRIBUTION

Docket File

NRC PDR

Local PDR

PD21 r/f

SVarga

DHagan

GLainas

ESylvester

OGC-B

PAnderson

EJordan

BGrimes

TBarnhart (8)

Wanda Jones

JPartlow

ACRS (10)

GPA/PA

ARM/LFMB

EButcher

RBernero

JPulsipher