

December 4, 1984

Docket Nos. 50-325/324

Mr. E. E. Utley  
Executive Vice President  
Carolina Power & Light Company  
Post Office Box 1551  
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment Nos. 79 and 106 to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal of June 6, 1984.

The amendments revise the Technical Specifications to more clearly define the operational conditions and the allowed use of the reactor mode switch by adding and revising footnotes in Table 1.2.

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Marshall Grotenhuis, Project Manager  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 79 to  
License No. DPR-71
2. Amendment No. 106 to  
License No. DPR-62
3. Safety Evaluation

cc w/enclosures:  
See next page

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Docket File

NRC PDR

Local PDR

ORB#2 Reading

DEisenhut

KPorter, RII

SNorris

MGrotenhuis

OELD

LJHarmon

ELJordan

PMcKee

TBarnhart (8)

WJones

DBrinkman

ACRS (10)

OPA, CMiles

RDiggs

Gray File

Extra - 5

JPartlow

DL:ORB#2  
SNorris:ajs  
11/14/84

DL:ORB#2  
MGrotenhuis  
11/14/84

DL:ORB#2  
DVassallo  
11/19/84

OELD  
11/21/84

DL:AD-OR  
Glainas  
11/14/84

Mr. E. E. Utley  
Carolina Power & Light Company  
Brunswick Steam Electric Plant, Units 1 and 2

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79  
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power & Light Company (the licensee) dated June 6, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 79, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 4, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 79

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Revise the Appendix A Technical Specifications as follows:

Remove

1-10

Insert

1-10

TABLE 1.2  
OPERATIONAL CONDITIONS

<u>OPERATIONAL CONDITIONS</u>	<u>MODE SWITCH POSITIONS</u>	<u>AVERAGE REACTOR COOLANT TEMPERATURE</u>
1. POWER OPERATION	Run	Any temperature
2. STARTUP	Startup/Hot Standby	Any temperature
3. HOT SHUTDOWN	Shutdown <sup>#,***</sup>	> 212°F
4. COLD SHUTDOWN	Shutdown <sup>##,##,***</sup>	≤ 212°F
5. REFUELING <sup>*</sup>	Shutdown or Refuel <sup>**,#</sup>	≤ 212°F

<sup>#</sup> The reactor mode switch may be placed in the Run or Startup/Hot Standby position to test the switch interlock functions provided that the control rods are verified to remain fully inserted by a second licensed operator or other technically qualified member of the unit technical staff.

<sup>##</sup> The reactor mode switch may be placed in the Refuel position while a single control rod drive is being removed from the reactor pressure vessel per Specification 3.9.10.1.

<sup>\*</sup> Fuel in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.

<sup>\*\*</sup> See Special Test Exceptions 3.10.1 and 3.10.3.

<sup>\*\*\*</sup> The reactor mode switch may be placed in the Refuel position while a single control rod is being recoupled provided that the one-rod-out interlock is OPERABLE.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106  
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power & Light Company (the licensee) dated June 6, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 106, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "D. Vassallo", written in a cursive style.

Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 4, 1984



ATTACHMENT TO LICENSE AMENDMENT NO. 106

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Revise the Appendix A Technical Specifications as follows:

Remove

1-11

Insert

1-11

TABLE 1.2

OPERATIONAL CONDITIONS

<u>OPERATIONAL CONDITIONS</u>	<u>MODE SWITCH POSITIONS</u>	<u>AVERAGE REACTOR COOLANT TEMPERATURE</u>
1. POWER OPERATION	Run	Any temperature
2. STARTUP	Startup/Hot Standby	Any temperature
3. HOT SHUTDOWN	Shutdown <sup>#</sup> ,***	> 212° F
4. COLD SHUTDOWN	Shutdown <sup>#</sup> ,##,***	≤ 212° F
5. REFUELING*	Shutdown or Refuel <sup>**</sup> ,#	≤ 212° F

# The reactor mode switch may be placed in the Run or Startup/Hot Standby position to test the switch interlock functions provided that the control rods are verified to remain fully inserted by a second licensed operator or other technically qualified member of the unit technical staff.

## The reactor mode switch may be placed in the Refuel position while a single control rod drive is being removed from the reactor pressure vessel per Specification 3.9.10.1.

\* Fuel in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.

\*\* See Special Test Exceptions 3.10.1 and 3.10.3.

\*\*\* The reactor mode switch may be placed in the Refuel position while a single control rod is being recoupled provided that the one-rod-out interlock is OPERABLE.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 79 TO FACILITY LICENSE NO. DPR-71 AND  
AMENDMENT NO. 106 TO FACILITY LICENSE NO. DPR-62  
CAROLINA POWER & LIGHT COMPANY  
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-325 AND 50-324

1.0 Introduction

By letter dated June 6, 1984 the Carolina Power & Light Company (CP&L/the licensee) submitted proposed changes to the Technical Specifications appended to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant (BSEP), Units 1 and 2. The proposed changes revise the Technical Specifications to more clearly define the operational conditions and the allowed use of the reactor mode switch by adding and revising footnotes in Table 1.2.

2.0 Evaluation

The licensee has submitted to the NRC a proposed revision to the Brunswick Technical Specifications. The proposed Technical Specification is delineated below:

Table 1.2 - Footnotes are added to allow for temporary placement of the Reactor Mode Switch (RMS) in the Run or Startup/Hot Standby positions to test the switch interlock functions. The purpose of this revision is to clarify the requirements for mode switch manipulations currently authorized in Technical Specifications. Footnotes are added to allow placement of the RMS in the refuel position when work on a single control rod is being performed. A more precise definition of the refueling operational condition is provided and the footnote format is changed. "Average Coolant Temperature" column title is changed to "Average Reactor Coolant Temperature" to conform to Standard Technical Specifications (STS).

There are presently three footnotes in Table 1.2. Two of those footnotes were combined into one and the third was rewritten for clarification. Three new footnotes are added which provide for Reactor Mode Switch placement while in the Run or Startup/Hot Standby position, the Refuel position while a single rod is being removed, and the Refuel position while a single rod is being uncoupled.

The footnotes added in this revision allow for temporary placement of the Reactor Mode Switch in the Run or Startup/Hot Standby positions to test the switch interlock function or placement of the RMS in the fuel position when

work on a single control rod is being performed. In each instance, measures required to ensure safety which are consistent with guidance provided in the STS are included.

The proposed revision clarifies the definitions pertaining to operational conditions and makes the BSEP Technical Specifications more closely resemble the Standard Technical Specifications.

Having reviewed the licensee's submittal consisting of the proposed Technical Specification change and the associated justification for the change, we have determined that this revision causes no significant increase in the probability or consequence of a previously analyzed accident nor a significant reduction in safety margin and is consistent with guidance provided in the Standard Technical Specifications.

### 3.0 Environmental Considerations

The amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 4.0 Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Porter, Region II

Dated: December 4, 1984