

Operated by Nuclear Management Company, LLC



December 21, 2001 NG-01-1428

Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Attn: Document Control Desk Mail Station 0-P1-17 Washington, DC 20555-0001

Subject:	Duane Arnold Energy Center
-	Docket No: 50-331
	Op. License No: DPR-49
	Single-Failure-Proof Status of Reactor Building Crane

References: 1. NG-96-1035, dated May 10, 1996. from J. Franz to NRC; IES Response to NRC Bulletin 96-02: Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor Core, or Over Safety-Related Equipment
2. Letter dated August 3, 2001, from B. Mozafari (NRC) to G. Van Middlesworth (NMC); Duane Arnold Energy Center - Single-Failure-Proof Status of Reactor Building Crane
3. NG-01-1029, dated August 31, 2001, from G. Van Middlesworth to NRC; Single-Failure-Proof Status of Reactor Building Crane

File: A-101a, T-31, SPF-164

In April of 1996, the NRC issued Bulletin 96-02, "Movement of Heavy Loads Over Spent Fuel. Over Fuel in the Reactor Core. or Over Safety-Related Equipment." The Bulletin requested that licensees review plans and capabilities for handling heavy loads while the reactor is at power in accordance with existing regulatory guidelines and licensing basis. The Duane Arnold Energy Center's (DAEC's) response to Bulletin 96-02 was provided by letter dated May 10, 1996 (Reference 1).

As discussed in that letter, the DAEC's review identified an issue involving the single-failureproof status of the reactor building crane. As described in more detail in Attachment 1, when the reactor building crane was upgraded in 1985, the NRC did not review the seismic analysis which was performed to verify that the crane would be capable of safely supporting its rated load during a seismic event.

By letter dated August 3, 2001 (Reference 2), the Staff requested that Nuclear Management Company, LLC (NMC) revise the DAEC Updated Final Safety Analysis Report (UFSAR) to clarify that the NRC had not endorsed the crane as single-failure-proof. The DAEC UFSAR was revised accordingly (Reference 3).

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It is desirable to resolve the single-failure-proof status of the DAEC reactor building crane well in advance of the implementation of dry spent fuel storage, currently anticipated in the Summer of 2003. In support of anticipated heavy load handling, additional seismic calculations have been performed regarding the reactor building crane. A summary of the assumptions and conclusions of these calculations is provided in Attachment 1. Attachment 2 contains portions of the seismic analysis for the Staff's review, as well as the "Static Load Test Procedure for New Reactor Building Crane," dated December 4, 1984.

NMC requests the Staff's review and approval of the seismic analysis prior to December 31, 2002, so that this issue may be resolved prior to implementation of dry spent fuel storage. Upon approval of the seismic analysis by the NRC, NMC will revise the DAEC UFSAR to reflect the Staff's review.

Should you have any questions regarding this matter, please contact this office.

Sincerely,

lun Middlewood

Gary Van Middlesworth DAEC Site Vice President

Attachments: 1. Summary of Analysis 2. Portions of Calculations and Static Load Test Procedure

cc: T. Vine C. Rushworth R. Anderson (NMC) B. Mozafari (NRC-NRR) J. Dyer (Region III) NRC Resident Office DOCU

Attachment 1 to NG-01-1428 Page 1 of 2

Summary of Analysis

In 1985, the DAEC reactor building crane was modified under 10 CFR 50.59. A new Ederer single-failure-proof crane trolley and main hoist system meeting the guidelines of Regulatory Guide 1.104, "Overhead Crane Handling Systems for Nuclear Power Plants," and NUREG-0554, "Single Failure Proof Cranes for Nuclear Power Plants," was installed. The design of the Ederer hoist and trolley system was evaluated in a Staff Safety Evaluation Report (SER) of the Generic Licensing Topical Report EDR-1, Rev. 3, for Ederer's Nuclear Safety-Related Extra Safety and Monitoring (X-SAM) Cranes, dated August 3, 1983.

The trolley system was installed on the existing bridge; the bridge itself was not replaced. A seismic analysis was performed to verify that the reactor building overhead crane would be capable of safely supporting its rated load during a seismic event after installation of the new, heavier, single-failure-proof trolley. This calculation concluded that the combined vertical and horizontal stresses developed in an operational basis earthquake (OBE) and a design basis earthquake (DBE) are within the allowable stress limits defined in Crane Manufacturers Association of America (CMAA) 70 - 1975 (Specification for Electric Overhead Traveling Cranes). This analysis was not reviewed by the NRC.

Calculation for Reactor Building Crane Girders

The purpose of the calculation is to check the design of the reactor building crane girders for the increased loadings imposed by the trolley upgrade. The calculation utilizes standard engineering practice and follows analysis methods described in previous DAEC calculations. Conservative inputs and assumptions are used throughout this calculation. Specific assumptions include:

- CMAA #70 1975 edition was used.
- UFSAR earthquake accelerations are used for the reactor building and runway.
- Vertical accelerations of 0.09g OBE and 0.18g DBE are used. The vertical seismic load combinations include the weight of the crane and the lifted load (100 tons).
- The horizontal accelerations used were 0.60g OBE and 1.20g DBE at the crane level. The lifted load and lower load block are assumed to be decoupled from the bridge and trolley with respect to horizontal earthquake accelerations.

The new analysis concluded that the existing crane girders are adequate. The girder combined stresses were shown to be less than allowable stresses. Deflections are less than allowable. Wheel loads are higher than recommended, possibly increasing wheel wear by a small amount: however, this is judged to be acceptable given the number of maximum rated lifts over the remaining plant life. Diaphragm spacing is greater than recommended (because of the increased trolley wheel loads), but is acceptable by similar reasoning to the wheel loads above. End truck stresses are less than allowable. The girder to end truck connections are acceptable.

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Calculation for Reactor Building Structure

The purpose of this calculation is to check the design of the reactor building structure for the new loadings imposed by the 100 ton capacity single-failure-proof reactor building crane. The calculation also evaluates the design condition of maximum lifted loads during a seismic event. The calculation utilizes standard engineering practice and follows analysis methods described in previous DAEC calculations. Conservative inputs and assumptions are used throughout this calculation. Specific assumptions include:

- Vertical accelerations of 0.04g OBE and 0.08g DBE are used. The vertical seismic load combinations include the weight of the crane and the lifted load (100 tons).
- The horizontal accelerations used were 0.35g OBE and .70g DBE at the crane level and .52g OBE and 1.04g DBE at the roof level. The lifted load and lower load block are assumed to be decoupled from the bridge and trolley with respect to horizontal earthquake accelerations.

The analysis concluded that the existing crane runway girders and rigid frame are adequate. The reactor building crane support structure is adequately designed for the increased weight of the replacement trolley and for all appropriate load combinations, including maximum lifted load plus seismic.

Attachment 2 to NG-01-1428

Portions of Calculations and Static Load Test Procedure

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DOCUMENT NUMBER: CAL-M01-273

Title : REACTOR BUILDING CRANE GIRDER CHECK

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Attachments

1. EDER	ER TROLLEY	WHEEL LOAD	DISTRIBUTION
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- 2. CENTERLINE GIRDER LOAD IDENTIFICATION
- 3. DAEC REACTOR BUILDING CRANE OPERATING INSTRUCTIONS
- 4. MHE (S. PARKHURST) REVIEW OF MAX. WHEEL LOADS

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Record of Changes

Rev. No.	Description Of Changes	Pages Revised	Pages Added	Pages Replaced

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1. OBJECTIVE OF CALCULATION

The purpose of this calculation is to check the design of the Reactor Building crane girders for the increased loadings imposed by the trolley upgrade installed in 1984. The upgrade replaced the original P&H Harnischfeger trolley with an Ederer XSAM single-failure-proof trolley, which weighs 20.800 pounds more than the original trolley. The original P&H trolley weighed 65,100 pounds (Ref. #3, pg. 11). S&W Report No. 12133.1004-S(C)-1, Rev. 1 (Ref. #1) reported a trolley weight of 82,000 pounds (an increase of 17,000 pounds), as indicated in several DAEC calculations. However Attachment #1 and Ederer Drawing A13151 indicate an as-built trolley weight of 85,898 pounds as used in this calculation, for an increase of 20,800 pounds from the P&H trolley.

The calculation is a follow-up effort to the S&W Report No. 12133.1004-S(C)-1, Rev. 1 (Ref. #1). The calculation provides a new and more detailed study of the structural adequacy of the Reactor Building crane girders for the increased weight (21 kips) of the trolley. The calculation also evaluates the design condition of maximum lifted loads during a seismic event.

The following specific points will be addressed by this calculation:

- Design check of crane girders (including rail) using increased trolley load of 20,800 lbs.
- Design check using revised trolley wheelbase geometry
- Design check for unequal wheel loads
- Design check for locations of wheel loads at worst-case positions along the girder
- Design check of welds at girder web to flanges
- Design check of diaphragms and stiffeners
- Provide reference to seismic documentation (UFSAR and specifications) in calculation

2. CALCULATION METHODS/ASSUMPTIONS

The calculation utilizes standard engineering practice and follows analysis methods described in previous DAEC calculations. Conservative inputs and assumptions are used throughout this calculation. Specific assumptions, are identified below:

- CMAA #70 1975 edition (Ref. #12) is to be used
- Vertical earthquake accelerations include the lifted load and the hook
- Horizontal earthquake accelerations exclude the lifted load and the hook
- The earthquake accelerations to be used are from References #14 & #15 and are specific to the crane girder. The UFSAR (Ref. #2) accelerations are used for the reactor building and runway only

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3. SOURCES OF DATA/EQUATIONS

Vertical impact is 15% of the rated load. Lateral loads are 2½% of live & dead loads. Deflection allowance is 0.00125 inches per inch of span.

The list of references used in this calculation is shown below in Section 5. This calculation uses vertical accelerations of 0.09g OBE and 0.18g DBE (Ref. #14 & #15). The vertical seismic load combinations include the weight of the crane and lifted load. The horizontal accelerations used were 0.60g OBE and 1.20g DBE at the crane level. Horizontal seismic loads include only the dead loads of the trolley and girders (i.e., not the lifted load or hook). The lifted load and hook in horizontal seismic load combinations are typically not included since they are free to sway.

4. CONCLUSIONS

The analysis of the existing crane girders were shown to be adequate as designed. The girder combined stresses were shown to be less than allowable stresses (see page 13). Deflections are less than allowable (see page 19). Wheel loads are greater than recommended, possibly increasing wheel wear by a minor amount. This is judged to be acceptable given the number of maximum rated lifts over the remaining plant life (see page 19). Diaphragm spacing is greater than recommended and diaphragm thickness is less than allowable (both because of the increased trolley wheel loads), but both are acceptable (see page 16) by similar reasoning to the wheel loads above. End truck stresses are less than allowable (see pages 20 & 21). The girder to end truck connections are acceptable.

5. REFERENCES

- 1. Phase I Structural Evaluation Report for DEAC Reactor Building Crane Bridge Girders. Duane Arnold Energy Center, 12133.1004-S(C)-1, Rev. 0, dated 06/04/01
- 2. DAEC UFSAR, Sections 2.5 Rev. 14, 3.7 Rev 14 and 3.8, Rev. 12
- 3. Bechtel DAEC Calculation 3-F-6, Rev. 0, dated 05/08/70, Crane Frame (Space Frame) Reactor Building
- 4. P&H DAEC Calculation M-023-051, Rev. 3, dated 11/14/84. Reactor Building Crane
- 5. Iowa Electric Light and Power Company DAEC Calculation C-85-19, Rev. 0, dated 07/18/85, Static Seismic Analysis of Rx Bldg. Overhead Crane
- 6. Iowa Electric Light and Power Company DAEC Calculation C-87-14, Rev. 0, dated 12/04/87, Girder Loading for Rx Bldg. Crane
- 7. Bechtel DAEC Drawing C-437, Rev. 4, R.B. Craneway Plan & Details
- 8. P&H Drawing 28A11849, sheets 1 & 2, dated 02/23/71
- 9. P&H Drawing 105A3310, Rev C (M-23-97 Sheets 1&2)
- 10. P&H Drawing 31A5826-5, Rev A (M23-98-1)
- 11. American Institute of Steel Construction Manual of Steel Construction. Allowable Stress Design. 9th edition
- 12. Crane Manufacturers Association of America CMAA Specification #70, 1975
- 13. John A. Blume & Associates Report DAEC Reactor Building Earthquake Analysis APED-A61-047, Rev. 1

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- 14. DAEC Specification 7884-M-23A, Rev. #2 (BECH-MRS-M23A), Technical Specification for the Modification to the Reactor Building Crane
- 15. DAEC Specification 7884-M-23.1, Specific Conditions for Rector Building Crane Modifications (Engineering Only)
- 16. Omar Blodgett Design of Welded Structures, 1966
- 17. William Weaver, Whiting Crane Handbook, 1979
- 18. P&H Drawing 29E5472. Rev A

6. DESIGN BASIS AND ASSUMPTIONS

- 1. Design method in general is based upon existing calculations (Ref. #4, #5 and #6) with allowable stresses from CMAA #70 (Ref. #12).
- 2. OBE and DBE design earthquake accelerations based on References #14 and #15, and are specific to the crane girders.
- 3. Impact is taken as 15% of rated capacity.
- 4. Lateral load is 2.5% of live load and crane bridge.
- 5. Trolley load distribution is based upon email dated 07/28/01 from Jim Nelson of Ederer to Stephen Parkhurst, Crane & Equipment Handling Specialist (Attachment #1 to this calculation).
- 6. E-60xx electrodes used for welding.

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7. Analysis

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Reactor Building Crane Support Structure Design Check

CALCULATION SUMMARY

OBJECTIVE OF CALCULATION:

The purpose of this calculation is to check the design of the Reactor Building structure for the new loadings imposed by the 100 ton capacity Reactor Building crane. The new loads are the result of increased weight associated with the replacement single-failure-proof trolley installed in 1986.

The calculation is a follow-up effort to the S & W Report No. 12133.1004-S(C) - 2, Rev. 0 (Ref.#1). The calculation provides a new and more detailed study of the structural adequacy of the Reactor Building crane runway and support structure, for the increased weight (21 kips) of the trolley. The calculation also evaluates the design condition of maximum lifted loads during a seismic event.

The following specific points will be addressed by this calculation:

- Design check of crane runway (including rail) and runway support structure using increased trolley load of 20,800 lbs.
- Design check using the revised trolley wheelbase geometry.
- Design check for unequal wheel loads.
- Design check for locations of wheel loads at worst-case positions along the runway airders.
- Design check of welds at runway girder brackets and bearing stiffeners due to increased loads.
- Provide reference to seismic documentation (UFSAR) in calculation.
- Re-compute seismic load combinations using vertical accelerations stated in UFSAR.
- Design check of the capability to transfer lateral loads from the crane to the runway girder.

CALCULATION METHOD/ASSUMPTIONS:

The calculation utilizes standard engineering practice and follows analysis methods described in previous DAEC calculations. Specific assumptions, as required, are identified within the body of the calculation. Conservative inputs and assumptions are used throughout this calculation.

The original design assumptions were used for this analysis. They were taken from Ref. #4 and are listed on page 7 of this calculation. The analysis method used for this calculation is both hand calculation and GT STRUDL Version 25. Additional assumptions and discussion of analysis methods are given on page 10 of this calculation.

SOURCES OF DATA/EQUATIONS:

The list of references used in this calculation is shown on page 7. This calculation uses vertical accelerations of .04g OBE and .08g DBE (Ref. 10, page 5). The vertical seismic load combinations include the weight of the crane and the lifted load. The horizontal accelerations used were .35g OBE and .70g DBE at the crane level and .52g OBE and 1.04g DBE at the roof level. Horizontal seismic loads include only the dead load of the crane (i.e., not the lifted load). The lifted load in

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horizontal seismic load combinations is typically not included due to the fact that the hanging load is free to sway.

CONCLUSIONS:

The analysis of the existing crane runway girders were shown to be adequate as designed. The combined stresses were shown to be less than the allowable stresses. See pages 41 - 43.

The rigid frame bents were then analyzed using the worst case loads from the runway analysis and were found to be adequate as designed, except for the DBE load combinations which were found to exceed the allowable stresses, when only one frame is considered. A new analysis was then performed which considered the contribution of the roof bracing and end walls. This analysis proved that the design is adequate for the DBE load cases as well. See pages 44 - 98.

The haunch at the top of the columns in the rigid frame was simplified for the analysis. This was considered a conservative approach, since the critical sections for moment were located in the transverse girder and the column base.

Connection details were then checked and found to be adequately designed. See pages 99 -106.

The Reactor Building crane support structure is adequately designed for the increased weight of the replacement trolley and for all appropriate load combinations, including maximum lifted load plus seismic.

The foundation design was not investigated, because the increase in column forces due to the increased trolley weight was not of significant magnitude to warrant a new analysis.

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References:

- Phase I Structural Evaluation Report for DAEC Reactor Building Crane Runway/Support 1. Structure, Duane Arnold Energy Center, 12133.1004-S(C) - 2, Rev. 0, dated 6/25/01.
- 2. DAEC UFSAR, Sections 2.5, 3.7, and 3.8, Rev. 13.
- 3. Phase I Structural Evaluation Report for DAEC Reactor Building Crane Girders, Duane Arnold Energy Center, 12133.1004-S(C) - 1, Rev. 0, dated 6/04/01.
- 4. Bechtel DAEC Calculation 3-F-6, dated 5-8-70, Crane Frame (Space Frame) R.B.
- 5. Bechtel DAEC Calculation 3-F-6, April, 1971, Supplemental Calculation for Rigid Frame.
- 6. Bechtel DAEC Calculation 3-F-8, dated 11/7/73, Reactor Building Overloading of Reactor Crane.
- 7. **Bechtel DAEC Drawings**
 - BECH-A008, Rev. 20, Arch. Roof Plan a.
 - BECH-A009, Rev. 20, Arch. North, South & Partial Exterior elevations b.
 - BECH-A010, Rev. 15, Arch. East, West & Partial Exterior Elevations C.
 - d. C-435, Rev. 3, R.B. Roof Framing Plan, Sections & Details
 - C-437, Rev. 4, R.B. Craneway Plan & Details e.
 - f. C-438, Rev. 2, R.B. Framing Elevations & Details Sheet #1
 - q. C-439, Rev. 1, R.B. Framing Sections & Details Sheet #2
 - C-442, Rev. 1, R.B. Rigid Frame Cross Section & Details h.
- 8. AISC Manual of Steel Construction, Sixth (1963) & Seventh (1970) editions.
- 9. AISC Manual of Steel Construction, Ninth (1989) edition.
- 10. DEAC Reactor Building Earthquake Analysis, by John A. Blume & Associates, Engineers, APED-A61-047, Rev. 1

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ORIGINAL DESIGN ASSUMPTIONS (Ref. #4, page 5) (comments in parentheses)

- 1. Impact will be considered for the design of the crane runway girders and the related connections but not for frame design.
- 2. Lifted load will not act simultaneously with Tornado and Earthquake. The (bridge and) trolley is in stored position. (no lateral force and impact)
- 3. One third of siding area (considered) left (intact) for tornado. No roof suction is considered for this case.
- 4. Earthquake force is assumed to concentrate at the rigid frame at two points. El. 876'-6" and El. 894'-8". (these elevations were modified slightly to coincide with as-built condition, see page 10, item 8 for details)
- 5. Roof loads are assumed to be uniformly distributed directly to the rigid frame. Purlins and other bracing members do not take external load in the computer analysis. -Purlins will be analyzed manually. (purlins were not in the scope of this calculation)
- 6. Assume that the roof girders are level. (actual roof slope was modeled in GTSTRUDL)
- 7. Ends of purlins and all bracing members will be released, hinged ends.
- 8. Assume that it is a space frame.
- 9. Wind force is assumed to be uniformly distributed directly to columns.
- 10. Assume that roof loads and parapet loads lumped at El. 894'-8". Column, siding, and crane lumped at El. 876-6". (elevations were changed slightly to match as-built elevations)

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DESIGN BASIS AND ASSUMPTIONS

- 1. DESIGN METHOD BASED IN GENERAL ON EXISTING CALCS PREPARED BY BECHTEL (REF. #4 THRU #6) ANY DEVIATIONS AND EXCEPTIONS WILL BE NOTED BELOW.
- OBE AND DEE DESIGN EARTHQUAKE HORIZONTAL AND VERTICAL Ζ. ACCELERATIONS BASED ON ATTACHMENT 3 OF REF. #1. THE UFSAR (REF. # 2) LISTS THE SAME VALUES.
- 3. FOR LATERAL CRANE IMPACT LOADS, WE USED 20% OF THE SUM OF THE WEIGHTS OF THE LIFTED LOAD AND CRANE TROLLEY, BUT EXCLUDING THE BRIDGE WEIGHT, IN ACCORDANCE WITH AISC, REF # 9.
- LIFTED LOAD OF 200 (100T) WAS NOT INCLUDED WHEN 4. CALCULATING OBE AND DBE HORIZONTAL FORCES. THE LIFTED LOAD WAS INCLUDED IN DETERMINING VERTICAL OBE AND DBE FORCES.
- 5. ALL LOADS AND COMBINATIONS OF LOADS WERE BASED ON THE ORIGINAL CALCULATIONS AND UESAR (REF. # 2). SEE PAGES /1 \$12 FOR DETAIL.
- 5. THE CRANE WHEEL GEOMETRY AND DISTRIBUTION OF WHEEL LOADS WAS ADJUSTED FROM THE ORIGINAL DESIGN TO MORE CLOSELY REFLECT THE ACTUAL INSTALLED CONCITIONS, USING THE EDERER TROLLEY (43 TON) AND ACTUAL WHEEL BASE DIMENSIONS. THIS DETAIL IS SHOWN ON PAGES 13\$14.
- 7. CALCULATED WHEEL LOADS WERE PLACED ON THE CRANE RUNWAY AT 3 FT. INTERVALS TO DETERMINE WORST POSSIBLE SEACTIONS. A GT STRUDL MODEL OF THE RUNWAY BEAM NAS CREATED TO FIND THE MAXIMUM REACTIONS AT THE LOCATIONS OF THE RIGIA FRAME RENTS.
- RIGID FRAME MODEL WAS CREATED =ROM BECHTEL DWGS. £ . REF # 7. ELEVATIONS ARE TOP OF STEEL FOR HORIZONTAL MEMBERS AND CENTERLINE FOR VERTICAL HEMBERS. ROOF SLOPE ("4" / FT.) WAS ALSO INCLUDED,

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BUBJECT / TITLE		QA CATEGORY	CODE CLASS
JAEL REACTOR 34	ILDING CRANE SUPPOI	27 7	
- DAVE DUFFE	(coord)		
JEW LOADS:	WHEEL WAD "/IND	ACT	
	1) = '55K/2		
GINDER		- 2.5	
IMPACT	$= 0.25 \times 27.5^{-} = 6$. 88 *	
TROLLEY	$= 27.2(95.42) \pm 15$	5,75(35,42) =	20.31K
· / - · · - · /	97 (2)	97 (2)	
	(2)		
IMPACT	$= 0.25 \times 20.31 =$	5.08-	
LIFTED	COAD = 50 (180.84)	= 46.6 K	
-	97 (2)		
	// (_)		
		- 14	
/MPACT	$= 0.45 \times 46.6 = 0.15 \times 10^{-10}$	1.65 -	
TOTAL = 27.	5 + 6.88 + 20.31 + 5.	08K + 46 6K +	ILEK
		, e : = - + ;	1.03
<u> </u>	· · · · · · · · · · · · · · · · · · ·		

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CAEC REACTOR BUILDUR CRAVE CUR	QA CATEGORY/CO	DE CLASS
CEACINE SUPPORT CICANE SUPP	-3 <u></u>	
CRANE WHEEL LOADS WEED ON		
CRANE WREEL COADS LUSED SOR	SEISMIC FORLES)	
NEGLECT CITERED COAD FOR HOILIZE	ONTAL EQ:	
$7 HeN R_{1} = \frac{27.2}{27.2} (47 - 1.58) + 15.75$	$(85.417) = 40.63^{\circ}$	
47 97		
	v	
$K_2 = 27.2 + 15.75 - 40.63 =$	2.32	
•		
SO GIRDER WT. = 220.1 /4T.	$rucrs = 55.03^k$	
40.43	40.63	
50.03	1 55.03	
95.66	95.66*	
$\wedge \bullet \wedge$	'	
$\Psi' \Psi = \Psi$	• U	
	4	•
47.33 47.83 47.83 47.83	^{47.83} ^⁴	
,		
WHEEL LOAD (W/0 IMPACT, UNLOADED ,	CRANE) = 47.83 K	
-)		
EQ-LATERAL LOADS! OBE = 0.352	= EL, 876.5'	
0		
H, = 47.83 (0.	35) = 16.74."	
	、 、	
$H_2 = 47.83 \left(\frac{2.32}{40.1} \right)$	(0.35) = 0.96 K	
DBE = 0.700 c	FI ETAS'	
$H_{1} = 47.83(0.7)$	$(0) = 33.48^{k}$	
$H_2 = 47.33/2.32$	$\frac{2}{2}(0.70) = 1.41K$	
40, 2	51(

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STONE & WEBSTER ENGINEERING CORPORATION CALCULATION SHEET J.O./W.O./CALCULATION NO. REVISION PAGE 12/33-55-1 45010 6ª 0 17 PREPARER / DATE REVIEWER / CHECKER / DATE INDEPENDENT REVIEWER /DATE M.N. SCACCO -124/01 Dut colilor R. MA IOIIOI SUBJECT / TITLE QA CATEGORY / CODE CLASS DAEL REACTOR BUILDING CRANE SUPPORT \mathcal{T} CRANE WHEEL LOADS (CONT.) VERTICAL LOADS (DUE TO SEISMIC) : (REF: PG14) OBE = 0.04 9 × LOADED CRANE "NO IMPACT $V_1 = 94.43 (0.04) = 3.78^{-10}$ $V_2 = 94.43 \left(\frac{2.32}{40.63}\right) (0.04) = 0.22^{K}$ DBE = 0.08 A × LOADED CRANE WIND IMPACT $V_1 = 94.43 (0.08) - 7.55^{k}$ $V_2 = 94.43 \left(\frac{2.32}{40.43}\right)(0.08) = 0.43^{k}$ PART I ANALYSIS OF CRANE RUNWAY GINDER

	J.O. /W.O. / CALCULATION NO.	REVISION	PAGE
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DAEC REACTOR BUI	LDING CRANE SUPPORT	-	T
			<u> </u>
ANALYSIS OF RU	WWAY BEAM (CONT.)	
$E_{4x} = 19.99$	45:		
			· · ·
Fby = 0.75 1	$F_{y} = 0.75(36) =$	27.0 KSi	
AXIAL FORCE ON (LONGITUDAL) =	THE RUNWAY BEAM 10% OF MAX, WHEEL	= TRACTIVE LOAD (W/ IMPA	Folce ct)
= 0	. 10× 118.1 × 4 WHE	ELS = 47.24	ĸ
$fa = \frac{P}{A} =$	$\frac{47.24}{75.5} = 0.624$	Esi	
MAXIMUM <u>Kl</u> =	$\frac{K_{Ly}}{r_{y}} = \frac{1.0(22.75)}{5.25}$	<u>(12)</u> = 52.0	
$\frac{kl/r}{Cc} = \frac{5}{12}$	$\frac{2 \cdot D}{6 \cdot l} = 0.41 \Rightarrow C_{l}$	a = 0.506 (AISC TABLE 3) REF. 9
Fa = 0.506 x	Fy = 0.506 (36)	= <u>18.17 KSI</u>	·
$\frac{KL_{X}}{T_{X}} = \frac{1.0(22.7)}{14.9}$	<u>75)(12) -</u> 18.26 5		
Fex = 338.62	Esi (AISC, TA	BLE 3 ; REF#9	?)
Fey = 55.23	$E_{\rm ref}$ $C_{\rm mx} = C$.my = 1.0 (Ca	ws.)
CHECK COMBINED	STRESSES		
$\frac{fa}{0.6F_{g}} + \frac{fbx}{Fox}$	$ \frac{\epsilon}{F_{by}} \leq 1. $	O (AISCHI	- z)
$\frac{6.626}{0.6(36)} + \frac{1}{1}$	$\frac{5.78}{9.99} + \frac{3.35}{21.6} \leq 1$. 0	
		4972 61	O LAK

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· _ · · · · · · · · · · · · · · · · · ·	J.U./W.G./CALGULATION NO.		REVISION	PAGE 17
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SUBJECT / TITLE	Juit - 1011101		A CATEGORY	/ CODE CLASS
DAEC BEACTOR BUILD	ING CRANE SUPPORT		7	
DACC ZEACIDE ISKIG				
ANALYSIS OF R	UNWAY BEAM (CONT	-)		
$\frac{f_a}{F_a} + \frac{Cmx}{(1-\frac{f_a}{F_{ex}})}$	$\frac{f_{bx}}{(f_{bx})} + \frac{Cm_y f_{by}}{(f_{ey})}F_{by}$	≤ /.	0 (A)	15C H1-1)
$= \frac{0.626}{18.17} + \frac{(1.0)(}{(1-\frac{0.62}{338})}$	(15.78) + $(1.0)(3.35)(1-\frac{0.426}{55.23})21.6$	<u> </u>	/. 0	
= 0.034 + 0.	791 + 0.157 =	0.9	82 4	1.0oz
ALTERNATIVELY; CI	HECK EQN. HI-3:			
$\frac{f_{c}}{F_{a}} + \frac{f_{bx}}{F_{bx}}$	$- + \frac{f_{by}}{F_{by}} \leq 1.0$	(Aiso	c 41-3)
= <u>0.626</u> 18.17	<u>15.78</u> + <u>3.35</u> <u>~</u> 19.99 + <u>21.6</u>	1.0		
= 0.034 + 0	.789 + 0.155 =	0.9	78 c	1.0 <u>0 K</u>
:. COMBINED SEC	CTION (W36×194 + W21	x62)/	S ADEQ	UATE.
CHECE SHEAR	STRESS:			
$V_{MAX} = 236$	$f_{V} = \frac{236}{36.98(0.77)} =$	8.40	KSÍ	
Fr = 0. + Fy	= 0.4×36 = 14.4	KSI	<u>.</u>	<u>K</u>
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M.N. SCACCO 8/03/01	1)ma 10/, /01	l	K. INA	10/1/01
SUBJECT/TITLE	A DALLES CHARLES	, -	QA CATEGORY	CODE CLASS
DAEC REACTOR 341	LOING CRANE SUPPORT		<u> </u>	
1	C			
	H = UNLO ADED			
CHECK RUNWAY B.	ERM FOR (V=LOADED CR	LANE +	EQ - 0.	BE :
SEE PG. 17 FOR	OBE LOADS (VERTI	'CAL)		
, PATIO - UNIDADE	0 CRANE - 144.75	(3,405	-
CATTO UTOCO				
LOADED C	LANE 337.3			
SO, MAX. MOMENT	(STRONG-AXIS) = 977 E	FTX 0	.405 =	<u>39.5.7 K-F</u>
			•	
MAX. MOMENT (WEAK	L-AXIS) = 395.7 × 0 <u>.</u>	<u>359</u> =	138.50	K-FT.
ADD' MONGHIE D	UE TO UERTICAL ED	Acces	(SPA TA	. L .
AUD C MOMENT DE		イビビン	CE ILA / 10	N .
$M_{\rm H} = 0.044 {\rm a} {\rm x}$	977.0 = 39.08 K	- F.T.		
1 1 1 1 1 1 1 1 1 1	(3957 +39.08)/12) _	7.02	KSI
$f_{\beta Y} = \frac{\eta_X + \eta_V}{2}$	= 13/3/7 = 2/100 //			
2×	142.13			
		- 0.	· -4	
$f_{py} = My =$	138.50(12) = 8.	37 /25	57 1	e - 0
- Sy	198.02			
Č V V V V V V V V V V V V V V V V V V V				
0.25 STRESS INC	LEASE ALLOWED -	By in	SPECTION	J, THIS
CHSE IS LESS (CLITICAL THAN LOAD	ED CR	ANE W/ ,,	MPACT.
	SH = UNLOADER	۲		
CHECK RUNWAY	BEAM FOR V=LOADED	CRANE	+ EQ-	DBE :
		<u></u>		
MAY HOMEOIT (S	TRONG - AKIS) = 3957	$(\zeta - F)$	- (SAME	AS ABOVE)
				, ,
HAVE HOUST (W	$E_{AK} = A_{V,S} = 2957 v$	n 70 G	= 176	99 K-ET
MAX, MOREN, C		<u></u> f		
	7977 = 71/1			
$M_V = 0.08 q^{-\chi}$	375,1 - 51,66 2-	· F (.		
(*	
$f_{1} = \frac{(395.7 + 1)}{(395.7 + 1)}$	(8.16)(12) = 7.65	=51		
74Z	.93			
		. /	. /	NO TRACTIVE
$f_{by} = 276.99$	$\frac{12}{2} = 16.79 - 51$, /	a = 0 (FORCE)
198.0	2			
		- 5.5 5	TRESS	INCREASE
fan' for	$f_{L_{\mu}} \leq 1.50$		ALLOWE	A _ 1
	+ $ -$	(AISC H	(1-3)
i ra i dx	~ o y	<u> </u>		
7.65 167	7	-		
	/, / 6 ~ /. 5	ې م	, OK	
17.71 41.6				
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STATIC LOAD TEST PROCEDURE FOR NEW REACTOR BUILDING CRANE

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NOTE: Before the static load test is performed, all inspections and non-load pre-operational tests shall be completed and documented.

1.0 SCOPE

Prior to declaring the crane operational, a static load test must be performed on the crane to verify its structural and component integrity. This test procedure was developed by Design Engineering with input from Ederer Inc. Procedure No. 250, Test Sections 250.14 and 250.15. Test Section 250.14 is designed to test the integrity of the bridge and trolley structures. Test Section 250.15 is designed to test the trolley and bridge components by moving the bridge and trolley while suspending a test load. Due to the configuration of the DAEC, Test Section 250.14 cannot be performed without bringing the test load to the refuel floor. Therefore, this procedure shall combine Test Sections 250.14 and 250.15 to minimize the movement of the test load.

During the load test the ambient temperature on the refuel floor must be lowered to $50^{\circ}F(+5^{\circ}, -0^{\circ})$ to fulfill a commitment made to the NRC. (Reference the DCP form, page 5.) All tests shall be performed under the supervision of the Ederer Field Engineer. The Responsible Construction Engineer (RCE) shall be responsible for the overall coordination of the testing. Operations shall be responsible for the actual operation of the crane.

2.0 EQUIPMENT REQUIRED

The equipment required for the static load testing shall include but not be limited to the following:

- A. Test platform for supporting test weights (see Attachment I for requirements).
- B. Assortment of test weights in arrangements of approximately 6.25, 50, 100, and 125 tons (see Attachment I for details).

3.0 ADDITIONAL RECOMMENDATIONS

A. To successfully satisfy the requirements of the Ederer procedures, the tests only need to be performed with a test load weight of 1.25 times the crane rating (6.25T, 125T). However, Design Engineering recommends that for the main hoist the test be run with increasing amounts of weight (50T, 100T, 125T) to provide greater assurance that the trolley and bridge will support the 125T load. This procedure has been developed based on this criteria. If DAEC personnel feel that only the 125T test is required, consult Design Engineering for resolution.

- B. During the load test, Design Engineering recommends that personnel traffic in the Reactor Building be kept to a minimum and personnel be restricted from the load path on floors below elevation 855'-0".
- C. Attachment 1 provides the various configurations of weights required to obtain the 6.25, 50, 100, and 125T test loads. Recommended methods for stacking the weights are also provided; however, the field may utilize other methods if desired.

4.0 LOAD TEST FOR MAIN HOIST

A. 50 Ton Test Load

1. Connect the main hoist hook to the 50 ton test load and raise the test load approximately 6" above the 757'-6" floor using the slowest hook speed. To satisfy the requirements of Ederer Procedure 250 (Attachment I to Index Item 3.02), Test Section 250.14.c and Test Section 250.15.b, the test load shall be suspended approximately 5 minutes to ensure that the brakes are able to hold the load. If the hoist is not able to lift and hold the test load, refer to Section 251.12 of Ederer Procedure 251 (Attachment II to Index Item 3.02) for adjustment. The RCE shall verify that the crane is able to hold the load and that any adjustments have been completed.

Star RCE/Date

2. Raise the test load approximately 3' off of the floor, hold the load and then lower load using the slowest hook speed. After load speed has stabilized, stop the test load and ensure the brake holds the load. The RCE shall verify that the brakes were able to stop and hold the test load.

Signed: RCE/Date

3. Raise the test load up to the refuel floor using the slowest hook speed and follow the load path shown on Fig. 1. After completing the load movement on the refuel floor, the test load shall be lowered back down the equipment hatch at the slowest hook speed and set on the 757'-6" elevation, in the location shown in Fig. 2. The RCE shall verify that during the lift and trolley/bridge movements, the crane operates smoothly and performs normally.

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Signed: RCE/Date

B. 100 Ton Test Load

ren :

1. Perform the test described in Section 4.A.1 above for a 100T test load. The RCE shall verify that the crane is able to hold the load and that any adjustments have been completed.

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Signed: RCE/Date

Perform the test described in Section 4.A.2 above for the 100T 2. test load. The RCE shall verify that the brakes were able to stop and hold the test load.

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1-10-25 RCE/Date

Perform the test described in Section 4.A.3 above for the 100T 3. test load. The RCE shall verify that during the lift and trolley/bridge movements, the crane operates smoothly and performs normally.

Signed:

- C. 125 Ton Test Load
 - 1. Perform the test described in Section 4.A.1 above for a 125T test load. The RCE shall verify that the crane is able to hold the load and that any adjustments have been completed. The RCE shall verify that the checklists and Test Sections 250.14 and 250.15 have been filled out.

/-/0-85 RCE/Date

Perform the test described in Section 4.A.2 above for the 125T 2. test load. The RCE shall verify that the brakes were able to stop and hold the test load. The RCE shall verify that the checklists in Test Sections 250.14 and 250.15 have been filled out.

~ 1-10-RS Signed: RCE/Date

3. Perform the test described in Section 4.A.3 above for the 125T test load. The RCE shall verify that during the lift and trolley/bridge movements, the crane operates smoothly and performs normally. The RCE shall verify that the checklists in Test Sections 250.14 and 250.15 have been filled out.

tory 1-10-85 Signed: RCE/Date

- 5.0 LOAD TEST FOR AUXILIARY HOIST
 - A. 5.25 Ton Test Load
 - 1. Connect the auxiliary hoist hook to the 6.25 ton test load. Raise the test load approximately 3 ft. off of the 757'-6" floor using the slowest hook speed and stop the load and hold for approximately 5 minutes. The RCE shall verify that the brakes are able to hold the load without lowering the load. The RCE shall

- 2. Perform the test described in Section 4.A.2 above for the 100T test load. The RCE shall verify that the brakes were able to stop and hold the test load.
 3. Signed: RCE/Date
 3. Perform the test described in Section 4.A.3 above for the 100T test load. The RCE shall verify that during the lift and trolley/bridge movements, the crane operates smoothly and performs normally.
 Signed: RCE/Date
 - C. 125 Ton Test Load

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1. Perform the test described in Section 4.A.1 above for a 125T test load. The RCE shall verify that the crane is able to hold the load and that any adjustments have been completed. The RCE shall verify that the checklists and Test Sections 250.14 and 250.15 have been filled out.

1/24/85 Bergman Signed: RCE/Date

 Perform the test described in Section 4.A.2 above for the 125T test load. The RCE shall verify that the brakes were able to stop and hold the test load. The RCE shall verify that the checklists in Test Sections 250.14 and 250.15 have been filled out.

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1/26/85 Signed: RCE/Date

3. Perform the test described in Section 4.A.3 above for the 125T test load. The RCE shall verify that during the lift and trolley/bridge movements, the crane operates smoothly and performs normally. The RCE shall verify that the checklists in Test Sections 250.14 and 250.15 have been filled out.

1/26/05 Bergman RCE/Date Signed:

5.0 LOAD TEST FOR AUXILIARY HOIST

- A. 5.25 Ton Test Load
 - 1. Connect the auxiliary hoist hook to the 6.25 ton test load. Raise the test load approximately 3 ft. off of the 757'-6" floor using the slowest hook speed and stop the load and hold for approximately 5 minutes. The RCE shall verify that the brakes are able to hold the load without lowering the load. The RCE shall

ensure that the checklists on Test Sections 250.14 and 250.15 have been filled out.

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12-30-84 Signed: RCE/Date

2. Lower the test load using the slowest hook speed. After the hook speed has stabilized, stop the test load and ensure the brakes hold the load. The RCE shall verify that the brakes were able to stop and hold the test load. The RCE shall also verify that the checklists in Test Sections 250.14 and 250.15 have been filled out.

-30-24 Signed: RCE/Date

NUTE: It is not necessary to raise the 6.25 ton test load to the refuel floor. Lifting the test load close to the 757'-6" floor will sufficiently test the auxiliary hoist components.

Attachment:

I Platform Fabrication/Testing and Test Weight Transportation/ Arrangement

Figures:

1 Static Load Test Path for the New Crane Trolley

2 Test Weight Location on 757'-6" Floor

Attachment I to Static Load Test Procedure

PLATFORM FABRICATION/TESTING AND TEST WEIGHT TRANSPORTATION/ARRANGEMENT

K FCR - 1294-1 REVO

The test weights and associated lifting bails will be delivered to the DAEC. (Reference DCP Index Item 8.19) All required equipment will be provided to transfer the weights from the yard (Reference DCP Index Item 8.20). The transference of the weights from the area outside the fence to the yard, and from the yard to the reactor building shall be coordinated by the RCE.

Note: One at the lifting bails and the set of the test of test

2. The lifting platform and lifting slings have been designed to support the 125% load with a factor of safety of 10 to 1 as is required by NUREG 0612. A calculation (DCP Index Item 5.03) defines the platform design. The lifting platform will be supplied and fabricated in accordance with DCP Index Item 8.2000 In accordance with ANSI Standard N14.6-1978, an acceptance test will be performed on the platform. The platform will be subjected to a load test equal to 150% of the maximum load to which the device will be subjected.

Prior to the acceptance test on the platform, QC shall perform NDE (Liquid Penetrant or Magnetic Particle) on all welds where practicle. If weld is inaccessible, a visual examination should be performed on the weld. Inspections shall be in accordance with procedures defined in the Special Process Procedures Manual 1500 series (latest revision). Location for the testing should be coordinated with Design Engineering. QC and the RCE shall verify that all welds have been inspected and are acceptable.

Signed: RCE/Date) 12.17.84 Signed: QC/Date

The load test will be conducted at FMC per DCP Index Item 8

The RCE shall coordinate the platform testing with QC and FMC. The RCE shall verify that the platform has been tested with a load 150% greater than the expected maximum load (125T) and has successfully passed the test.

Fall 12-30-84

Signor: RCE/Date

Subsequent to testing, QC shall perform NDE (Liquid Penetrant or Magnetic Particle) on all welds where practicle to verify their integrity. If a weld is inaccessible, a visual examination should be performed on the weld. The examinations shall be in accordance with procedures defined in the Special Process Procedures Manual 1500 Series (latest revision) QC and the RCE shall verify that all welds have been inspected and are acceptable.

12-19-84

Signed: QC/Date

1-9-85 Signed: RCE/Date

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The slings for the platform will be furnished in accordance with DCP Index Item 8.10. Note that the platform will be used for the 125% load test and the 100% operational test for both the main and auxiliary hoist.

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- The allowable floor loading on the Reactor Building 757'-6" level in the 3. vicinity of the Equipment Hatch has been calculated (OCP Index Item 5.04). The floor can support the testing platform and weights in the location shown in attached Figure 2. The RCE shall ensure that the test weights and platform are located in this area during the 125% load test and the 100% operational test.
- 4. The recommended stacking sequences for the 5, 6.25, 50, 100 and 125 ton loads are listed below. It may be necessary for the RCE to coordinate the use of the tuning weights to obtain a more representative test weight. (Note: The actual weights are approximate due to the size of the individual test weights.)
 - A. Main Hoist

Stacking



50 Ton Load



B. Auxiliary Hoist

6.25 Ton







5. After the load and operational testing has been satisfactorily completed, the test weights shall be removed from the reactor building and placed in the yard. The weights and the bails will be returned to FMC at a later date per DCP Index Item 8.20. The RCE shall be responsible for coordinating the removal of the weights from the DAEC.

A FCR-1294.3 REV 0 # FOR THE AUXLLIARY LOAD FEST, SLINGS FROM THE DAEC SUPPLY SHOULD KE USED. THE FOLLSWING SLINGS ARE RECOMMENDED; SLING N'S BIZ; 213; 424 AND 425. (STORED IN TURGINE AREA) EACH SLING IS 3/4" JIA, 20-3" LONG AND RATED FOR ANTONS. OTHER SETS OF SLINGS OF AT LEAST 12.5 TONS OR GLEATER CAPACITY MAY BE USED IF ABOVE SLIPSS ARE NOT AWAILABLE





