

September 22, 1984

Docket Nos. 50-325/324

Mr. E. E. Utley
Executive Vice President
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment Nos. 76 and 102 to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the technical specifications in response to your submittal of May 10, 1984 as supplemented June 18, 1984.

The proposed amendments would revise the Technical Specifications 4.4.3.1.b and 4.4.3.2.a to incorporate changes in instrument tag numbers. The balance of the changes are editorial revisions to make nomenclature consistent with the Standard Technical Specifications and to correct three typographical errors.

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 76 to License No. DPR-71
2. Amendment No. 102 to License No. DPR-62
3. Safety Evaluation

cc w/enclosures:
See next page

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Extra - 5
MSinkule, Reg II

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09/15/84

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08/16/84

DL:ORB#2
DVassallo
08/16/84

OELD
09/1/84

DL:AB-OR
GLainas
09/17/84

[Handwritten signatures and dates]
09/10/84

Mr. E. E. Utley
Carolina Power & Light Company
Brunswick Steam Electric Plant, Units 1 and 2

cc:

Richard E. Jones, Esquire
Carolina Power & Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

George F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Mr. Charles R. Dietz
Plant Manager
Post Office Box 458
Southport, North Carolina 28461

Mr. Franky Thomas, Chairman
Board of Commissioners
Post Office Box 249
Bolivia, North Carolina 28422

Mrs. Chrys Baggett
State Clearinghouse
Budget and Management
116 West Jones Street
Raleigh, North Carolina 27603

U. S. Environmental Protection
Agency
Region IV Office
Regional Radiation Representative
345 Courtland Street, N. W.
Atlanta, Georgia 30308

Resident Inspector
U. S. Nuclear Regulatory Commission
Star Route 1
Post Office Box 208
Southport, North Carolina 28461

James P. O'Reilly
Regional Administrator
Region II Office
U. S. Nuclear Regulatory Commission
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Dayne H. Brown, Chief
Radiation Protection Branch
Division of Facility Services
Department of Human Resources
Post Office Box 12200
Raleigh, North Carolina 27605



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 76
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated May 10, 1984, as supplemented June 18, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

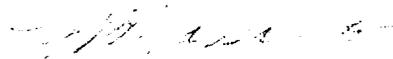
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2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 76, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: September 22, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 76

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Revise the Appendix A Technical Specifications as follows:

Remove

3/4 1-12

3/4 3-42

3/4 4-5

3/4 4-6

3/4 6-25

3/4 6-26

Insert

3/4 1-12

3/4 3-42

3/4 4-5

3/4 4-6

3/4 6-25

3/4 6-26

REACTIVITY CONTROL SYSTEMS

SURVEILLANCE REQUIREMENTS

4.1.3.7.1 The control rod reed switch position indicators shall be determined OPERABLE by verifying:

- a. At least once per 24 hours, that the position of the control rod is indicated,
- b. That the indicated control rod position changes during the movement of the control rod when performing Surveillance Requirement 4.1.3.1.2, and
- c. That the control rod reed switch position indicator corresponds to the control rod position indicated by the "full-out" reed switches when performing Surveillance Requirement 4.1.3.6.b.

4.1.3.7.2 When the RSCS is required to be OPERABLE, the position and bypassing of control rods with inoperable "Full-in" or "Full-out" reed switch position indication shall be verified by a second licensed operator or other qualified member of the technical staff.

TABLE 3.3.4-2

CONTROL ROD WITHDRAWAL BLOCK INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>APRM (C51-APRM-CH. A,B,C,D,E,F)</u>		
a. Upscale (Flow Biased)	$\leq (0.66W + 42\%) \frac{T^*}{MTPF}$	$\leq (0.66W + 42\%) \frac{T^*}{MTPF}$
b. Inoperative	NA	NA
c. Downscale	$> 3/125$ of full scale	$> 3/125$ of full scale
d. Upscale (Fixed)	$\leq 12\%$ of RATED THERMAL POWER	$\leq 12\%$ of RATED THERMAL POWER
2. <u>ROD BLOCK MONITOR (C51-RBM-CH.A,B)</u>		
a. Upscale	$\leq (0.66W + 41\%) \frac{T^*}{MTPF}$	$\leq (0.66W + 41\%) \frac{T^*}{MTPF}$
b. Inoperative	NA	NA
c. Downscale	$> 3/125$ of full scale	$> 3/125$ of full scale
3. <u>SOURCE RANGE MONITORS (C51-SRM-K600A,B,C,D)</u>		
a. Detector not full in	NA	NA
b. Upscale	$\leq 1 \times 10^5$ cps	$\leq 1 \times 10^5$ cps
c. Inoperative	NA	NA
d. Downscale	≥ 3 cps	≥ 3 cps
4. <u>INTERMEDIATE RANGE MONITORS (C51-IRM-K601A,B,C,D,E,F,G,H)</u>		
a. Detector not full in	NA	NA
b. Upscale	$\leq 108/125$ of full scale	$\leq 108/125$ of full scale
c. Inoperative	NA	NA
d. Downscale	$\geq 3/125$ of full scale	$\geq 3/125$ of full scale
5. <u>SCRAM DISCHARGE VOLUME (C11-LSH-N013E)</u>		
a. Water Level - High	≤ 73 gallons	≤ 73 gallons

*T=2.43 for 8x8 fuel
T=2.39 for 8x8R fuel
T=2.39 for P8x8R fuel

REACTOR COOLANT SYSTEM3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGELEAKAGE DETECTION SYSTEMSLIMITING CONDITION FOR OPERATION

3.4.3.1 The following reactor coolant system leakage detection systems shall be OPERABLE:

- a. The primary containment atmosphere particulate radioactivity monitoring system,*
- b. The primary containment sump flow integrating system, and
- c. The primary containment gaseous radioactivity monitoring system.*

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With only two of the above required leakage detection systems OPERABLE, operation may continue for up to 31 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when the required gaseous and/or particulate radioactive monitoring system is inoperable; otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.1 The reactor coolant system leakage detection systems shall be demonstrated OPERABLE by:

- a. Primary containment atmosphere particulate and gaseous radioactivity monitoring systems-performance of a CHANNEL CHECK at least once per 12 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days, and a CHANNEL CALIBRATION at least once per 18 months. (CAC-AQH-1260-1,2,3; CAC-AQH-1262-1,2,3; CAC-AQH-1261-1,2,3)
- b. Containment sump flow integrating systems-performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and of a CHANNEL CALIBRATION at least once per 18 months. (G16-FQ-K603; G16-FYQ-K603; G16-FQ-K601; G16-FYQ-K601; G16-FY-K602; G16-FY-K600; G16-FT-NO13; G16-FT-NO03)

*The system is OPERABLE if one channel is OPERABLE.

REACTOR COOLANT SYSTEMOPERATIONAL LEAKAGELIMITING CONDITION FOR OPERATION

3.4.3.2 Reactor coolant system leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE.
- b. 5 gpm UNIDENTIFIED LEAKAGE averaged over any 24-hour period.
- c. 25 gpm total leakage averaged over any 24-hour period.
- d. 2 gpm increase in UNIDENTIFIED LEAKAGE within any 24-hour period except for the first 24 hours of reactor startup commencing with entry into OPERATIONAL CONDITION 2.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With any reactor coolant system leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within the limits within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.2 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

- a. Monitoring the drywell and equipment drain sump flow rates at least once per 24 hours, and (G16-FQ-K603; G16-FYQ-K603; G16-FQ-K601; G16-FYQ-K601; G16-FY-K602; G16-FY-K600; G16-FT-NO13; G16-FT-NO03).
- b. Monitoring the primary containment atmosphere particulate and gaseous radioactivity at least once per 24 hours. (CAC-AQH-1260-1,2,3; CAC-AQH-1262-1,2,3 and CAC-AQH-1261-1,2,3).

CONTAINMENT SYSTEMS

3/4.6.6 CONTAINMENT ATMOSPHERE CONTROL

STANDBY GAS TREATMENT SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.6.1 Two independent standby gas treatment subsystems shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 5, and *.

ACTION:

- a. With one standby gas treatment subsystem inoperable:
 - 1. In OPERATIONAL CONDITION 1, 2, or 3, restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - 2. In OPERATIONAL CONDITION 5 or *, restore the inoperable subsystem to OPERABLE status within 31 days or suspend irradiated fuel handling in the secondary containment, CORE ALTERATIONS, or operations that could reduce the SHUTDOWN MARGIN. The provisions of Specification 3.0.3 are not applicable.
- b. With both standby gas treatment subsystems inoperable;
 - 1. In OPERATIONAL CONDITION 1, 2, or 3, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
 - 2. In OPERATIONAL CONDITION 5 or *, suspend all irradiated fuel handling in the secondary containment, CORE ALTERATIONS, or operations that could reduce the SHUTDOWN MARGIN. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.6.6.1 Each standby gas treatment subsystem shall be demonstrated OPERABLE:
- a. At least once per 31 days by initiating, from the control room, flow through the HEPA filters and charcoal absorbers, and verifying that the subsystem operates for at least 10 hours with the heaters on automatic control.

*When irradiated fuel is being handled in the secondary containment.

CONTAINMENT SYSTEMSSURVEILLANCE REQUIREMENTS (Continued)

- b. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal absorber housings, or (2) following painting, fire or chemical release in any ventilation zone, communicating with the system by:
1. Verifying that the cleanup system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a., C.5.c, and C.5.d of Regulatory Guide 1.52, Revision 1, July 1976, and the system flow rate is 3000 cfm + 10%.
 2. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 1, July 1976, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 1, July 1976.
 3. Verifying a system flow rate of 3000 cfm + 10% during system operation when tested in accordance with ANSI N510-1975.
- c. After every 720 hours of charcoal absorber operation by verifying within 31 days after removal that a laboratory analysis of representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 1, July 1976, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 1, July 1976.
- d. At least once per 18 months by:
1. Verifying that the pressure drop across the combined HEPA filters and charcoal absorber banks is less than 8.5 inches Water Gauge while operating the filter train at a flow rate of 3000 cfm + 10%.
 - 2.* Verifying that the filter train starts on each secondary containment isolation test signal.
 3. Verifying that the heaters will dissipate at least 15.2 kw when tested in accordance with ANSI N510-1975.

*For the performance of this surveillance scheduled to be completed by February 25, 1981, a onetime-only exemption is allowed to extend this surveillance until "before the completion of the Spring 1981 outage," scheduled to commence in March, 1981.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 102
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated May 10, 1984, as supplemented June 18, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 102, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: September 22, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 102

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Revise the Appendix A Technical Specifications as follows:

<u>Remove</u>	<u>Insert</u>
3/4 1-12	3/4 1-12
3/4 4-5	3/4 4-5
3/4 4-6	3/4 4-6
3/4 6-25	3/4 6-25
3/4 6-26	3/4 6-26

REACTIVITY CONTROL SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

- b. In CONDITION 5* with a withdrawn control rod reed switch position indicator inoperable, fully insert the withdrawn control rod. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.1.3.7.1 The control rod reed switch position indicators shall be determined OPERABLE by verifying:

- a. At least once per 24 hours, that the position of the control rod is indicated,
- b. That the indicated control rod position changes during the movement of the control rod when performing Surveillance Requirement 4.1.3.1.2, and
- c. That the control rod reed switch position indicator corresponds to the control rod position indicated by the "Full-out" reed switches when performing Surveillance Requirement 4.1.3.6.b.

4.1.3.7.2 When the RSCS is required to be OPERABLE, the position and bypassing of control rods with inoperable "Full-in" or "Full-out" reed switch position indication shall be verified by a second licensed operator or other qualified member of the technical staff.

*At least each withdrawn control rod. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

REACTOR COOLANT SYSTEM3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGELEAKAGE DETECTION SYSTEMSLIMITING CONDITION FOR OPERATION

3.4.3.1 The following reactor coolant system leakage detection systems shall be OPERABLE:

- a. The primary containment atmosphere particulate radioactivity monitoring system,*
- b. The primary containment sump flow integrating system, and
- c. The primary containment gaseous radioactivity monitoring system.*

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With only two of the above required leakage detection systems OPERABLE, operation may continue for up to 31 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when the required gaseous and/or particulate radioactive monitoring system is inoperable; otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.1 The reactor coolant system leakage detection systems shall be demonstrated OPERABLE by:

- a. Primary containment atmosphere particulate and gaseous radioactivity monitoring systems-performance of a CHANNEL CHECK at least once per 12 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days, and a CHANNEL CALIBRATION at least once per 18 months. (CAC-AQH-1260-1,2,3; CAC-AQH-1262-1,2,3; CAC-AQH-1261-1,2,3)
- b. Containment sump flow integrating systems-performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and of a CHANNEL CALIBRATION at least once per 18 months. (G16-FQ-K603; G16-FYQ-K603; G16-FQ-K601; G16-FYQ-K601; G16-FY-K602; G16-FY-K600; G16-FT-N013; G16-FT-N003)

*The system is OPERABLE if one channel is OPERABLE.

REACTOR COOLANT SYSTEMOPERATIONAL LEAKAGELIMITING CONDITION FOR OPERATION

3.4.3.2 Reactor coolant system leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE.
- b. 5 gpm UNIDENTIFIED LEAKAGE averaged over any 24-hour period.
- c. 25 gpm total leakage averaged over any 24-hour period.
- d. 2 gpm increase in UNIDENTIFIED LEAKAGE within any 24-hour period except for the first 24 hours of reactor startup commencing with entry into OPERATIONAL CONDITION 2.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With any reactor coolant system leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within the limits within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.2 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

- a. Monitoring the drywell and equipment drain sump flow rates at least once per 24 hours, and (G16-FQ-K603; G16-FYQ-K603; G16-FQ-K601; G16-FYQ-K601; G16-FY-K602; G16-FY-K600; G16-FT-NO13; G16-FT-NO03).
- b. Monitoring the primary containment atmosphere particulate and gaseous radioactivity at least once per 24 hours. (CAC-AQH-1260-1,2,3; CAC-AQH-1262-1,2,3 and CAC-AQH-1261-1,2,3).

CONTAINMENT SYSTEMS

3/4.6.6 CONTAINMENT ATMOSPHERE CONTROL

STANDBY GAS TREATMENT SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.6.1 Two independent standby gas treatment subsystems shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 5, and *.

ACTION:

- a. With one standby gas treatment subsystem inoperable:
 - 1. In OPERATIONAL CONDITION 1, 2, or 3, restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - 2. In OPERATIONAL CONDITION 5 or *, restore the inoperable subsystem to OPERABLE status within 31 days or suspend irradiated fuel handling in the secondary containment, CORE ALTERATIONS, or operations that could reduce the SHUTDOWN MARGIN. The provisions of Specification 3.0.3 are not applicable.
- b. With both standby gas treatment subsystems inoperable;
 - 1. In OPERATIONAL CONDITION 1, 2, or 3, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
 - 2. In OPERATIONAL CONDITION 5 or *, suspend all irradiated fuel handling in the secondary containment, CORE ALTERATIONS, or operations that could reduce the SHUTDOWN MARGIN. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.6.6.1 Each standby gas treatment subsystem shall be demonstrated OPERABLE:
- a. At least once per 31 days by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the subsystem operates for at least 10 hours with the heaters on automatic control.

*When irradiated fuel is being handled in the secondary containment.

CONTAINMENT SYSTEMSSURVEILLANCE REQUIREMENTS (Continued)

- b. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire, or chemical release in any ventilation zone communicating with the system by:
1. Verifying that the cleanup system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c, and C.5.d of Regulatory Guide 1.52, Revision 1, July 1976, and the system flow rate is 3000 cfm \pm 10%.
 2. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 1, July 1976, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 1, July 1976.
 3. Verifying a system flow rate of 3000 cfm \pm 10% during system operation when tested in accordance with ANSI N510-1975.
- c. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 1, July 1976, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 1, July 1976.
- d. At least once per 18 months by:
1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 8.5 inches Water Gauge while operating the filter train at a flow rate of 3000 cfm \pm 10%.
 2. Verifying that the filter train starts on each secondary containment isolation test signal.
 3. Verifying that the heaters will dissipate at least 15.2 kw when tested in accordance with ANSI N510-1975.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555



SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 76 TO FACILITY LICENSE NO. DPR-71 AND
AMENDMENT NO. 102 TO FACILITY LICENSE NO. DPR-62
CAROLINA POWER & LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
DOCKET NOS. 50-325 AND 50-324

1.0 Introduction

By letter dated May 10, 1984 as supplemented June 18, 1984 the Carolina Power & Light Company (the licensee) submitted proposed changes to the Technical Specifications appended to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant (BSEP). Units 1 and 2. The proposed amendments would revise the Technical Specifications (TS) 4.4.3.1.b and 4.4.3.2a to incorporate changes in instrument tag numbers. The integrated drywell floor equipment drain leakage detection system has been replaced with a system consisting of two separate components. Separate instrument numbers are needed for each components, which results in new instrument numbers to be added. The balance of the changes are editorial revisions to make nomenclature consistent with the Standard Technical Specifications for BWR, NUREG-0123, Rev. 3 and to correct three typographical errors. The proposed changes are administrative and editorial revisions to Brunswick Technical Specifications 4.1.3.7, 3.3.4, 3.6.6.1, 4.6.6.1, 3.4.3.1, 4.4.3.1, 3.4.3.2 and 4.4.3.2.

2.0 Evaluation:

The proposed administrative and editorial revisions to Brunswick Technical Specifications are discussed below:

Technical Specification 4.1.3.7 is corrected to refer to Technical Specification 4.1.3.1.2 for surveillance requirements, not Technical Specification 4.1.3.1.

Technical Specification 3.3.4 is corrected to reference the proper table. Table number 3.3.4-1 changed to 3.3.4-2 (Unit 1 only).

Technical Specification 3/4.6.6.1 is corrected to refer to the proper Regulatory Guide and to make it more closely conform to the Standard Technical Specifications. In Unit 1 Technical Specifications an expired one-time exemption to the surveillance interval is deleted.

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PDR ADOCK 05000324
P PDR

Technical Specification 3/4.4.3.1 is revised to make the nomenclature consistent with BWR standard Technical Specifications, to incorporate changes in instrument tag numbers and to correct incorrect instrument numbers.

Technical Specification 3/4.4.3.2 is revised to make the nomenclature consistent with standard Technical Specifications, for BWRs, NUREG-0123, Rev. 3 to incorporate changes in instrument tag numbers and to correct incorrect instrument number.

We have reviewed the licensee's submittal consisting of the proposed Technical Specification changes and the associated justification for their change and we have determined that the proposed changes are administrative and editorial and that safe operation can be accomplished with these changes. Based on our review we conclude that the proposed changes to the Technical Specifications are acceptable.

3.0 Environmental Considerations

The amendments involve changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

4.0 Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Sinkule, Region II

Dated: September 22, 1984