



June 26, 1984

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Docket Nos. 50-325/324

Mr. E. E. Utley Executive Vice President Carolina Power & Light Company Post Office Box 1551 Raleigh, North Carolina 27602

Dear Mr. Utley:

SUBJECT: CORRECTED T/S PAGES

Re: Brunswick Steam Electric Plant, Units 1 and 2

On May 22, 1984 we issued Amendment No. 97 on Unit 2. On June 5, 1984 we issued Amendment No. 71 on Unit 1. Page 3/4 3-24A of Amendment No. 97 and page 5-1 of Amendment No. 71 were not properly corrected to allow for previous amendments to the same pages. We have enclosed the correct pages. Please correct your Technical Specifications accordingly. We regret any inconvenience this error may have caused you.

Sincerely,

Original signed by/

Domenic B. Vassallo, Chief Operating Reactors Branch #2 Division of Licensing

Enclosures: As stated

cc w/enclosures: See next page

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NRC FORM 318 (10/80) NRCM 0240			OFFICIAL RECORD COPY			☆ U.S. GPO 1983-400-247	

Mr. E. E. Utley Carolina Power & Light Company Brunswick Steam Electric Plant, Units 1 and 2

cc:

G,

Richard E. Jones, Esquire Carolina Power & Light Company 336 Fayetteville Street Raleigh, North Carolina 27602

George F. Trowbridge, Esquire Shaw, Pittman, Potts and Trowbridge 1800 M Street, N. W. Washington, D. C. 20036

Mr. Charles R. Dietz Plant Manager Post Office Box 458 Southport, North Carolina 28461

Mr. Franky Thomas, Chairman Board of Commissioners Post Office Box 249 Bolivia, North Carolina 28422

Mrs. Chrys Baggett State Clearinghouse Budget and Management 116 West Jones Street Raleigh, North Carolina 27603

U. S. Environmental Protection Agency Region IV Office Regional Radiation Representative 345 Courtland Street, N. W. Atlanta, Georgia 30308

Resident Inspector U. S. Nuclear Regulatory Commission Star Route 1 Post Office Box 208 Southport, North Carolina 28461 James P. O'Reilly Regional Administrator Region II Office U. S. Nuclear Regulatory Commission 101 Marietta Street, Suite 3100 Atlanta, Georgia 30303

Dayne H. Brown, Chief Radiation Protection Branch Division of Facility Services Department of Human Resources Post Office Box 12200 Raleigh, North Carolina 27605

5.0 DESIGN FEATURES

5.1 SITE

EXCLUSION AREA

5.1.1 The exclusion area shall be as shown in Figure 5.1.1-1.

LOW POPULATION ZONE

5.1.2 The low population zone shall be as shown in Figure 5.1.2-1, based on the information given in Section 2.2 of the FSAR.

SITE BOUNDARY

5.1.3 The SITE BOUNDARY shall be as a shown in Figure 5.1.3-1. For the purpose of effluent release calculations, the boundary for atmospheric releases is the SITE BOUNDARY and the boundary for liquid releases is the SITE BOUNDARY prior to dilution in the Atlantic Ocean.

5.2 CONTAINMENT

CONFIGURATION

5.2.1 The PRIMARY CONTAINMENT is a steel-lined reinforced concrete structure composed of a series of vertical right cylinders and truncated cones which form a drywell. This drywell is attached to a suppression chamber through a series of vents. The suppression chamber is a concrete steel-lined pressure vessel in the shape of a torus. The primary containment has a minimum free air volume of (288,000) cubic feet.

DESIGN TEMPERATURE AND PRESSURE

- 5.2.2 The primary containment is designed and shall be maintained for:
 - a. Maximum internal pressure 62 psig.
 - b. Maximum internal temperature: drywell 300°F. suppression chamber 200°F.
 - c. Maximum external pressure 2 psig.

5.3 REACTOR CORE

FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 560 fuel assemblies, with each 8 x 8 fuel assembly containing 63 fuel rods and each 8 x 8R fuel assembly containing 62 fuel rods. All fuel rods shall be cladded with Zircaloy 2. Each fuel rod shall have a nominal active fuel length of 146 inches for 8 x 8 fuel and 150 inches for 8 x 8R fuel.

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BRUNSWICK - UNIT 1

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

TRIP FUNCTION AND INSTRUMENT NUMBER RESPONSE	TIME (Seconds)#
b. Reactor Core Isolation Cooling System Isolation	
<pre>l. RCIC Steam Line Flow - High (E51-PDT-N017; E51-PDT-N018)</pre>	<u><</u> 13 ^(a) ###
(E51-PDTS-N017-2; E51-PDTS-N018-2)	
2. RCIC Steam Line High Flow Time Delay Relay (E51-TDR-K32; E51-TDR-K12)	NA
3. RCIC Steam Supply Pressure - Low (E51-PS-N019A,B,C,D)	NA
<pre>4. RCIC Steam Line Tunnel Temp - High (E51-TS-3319; E51-TS-3320; E51-TS-3321; E51-TS-3322; E51-TS-3323; E51-TS-3355; E51-TS-3487)</pre>	NA
5. Bus Power Monitor (E51-K42 and E51-K43)	NA
6. RCIC Turbine Exhaust Diaphram Pressure - High (E51-PS-NO12A,B,C,D)	NA
7. RCIC Steam Line Ambient Temperature - High (E51-TS-N603A,B)	NA
8. RCIC Steam Line Area∆ Temp - High (E51-dTS-N604A,B)	NA
9. Emergency Area Cooler Temperature - High (E51-TS-N602A,B)	NA
10. RCIC Equipment Room∆ Temp - High (E51-dTS-N601A,B)	NA
5. SHUTDOWN COOLING SYSTEM ISOLATION	
a. Reactor Vessel Water Level - Low, Level 1 N (B21-LT-N017A-1,B-1,C-1,D-1) (B21-LTM-N017A-1,B-1,C-1,D-1)	ЛА
b. Reactor Steam Dome Pressure - High N (B32-PS-N018A,B)	IA

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