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10 CFR 50.90

December 20, 2001
2130-01-20259

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Subject: Oyster Creek Generating Station
Docket No. 50-219
Facility Operating License No. DPR-16
Technical Specification Change Request No. 285
Replacement Technical Specification Pages

Reference: AmerGen Letter No. 2130-00-20314 dated December 29, 2000, "Technical Specification Change Request No. 285 - Offsite Power Sources"

In the Reference, AmerGen Energy Company, LLC (AmerGen) requested a change to the Section 3.7 Technical Specifications contained in Appendix A to the Oyster Creek Facility Operating License. A change was proposed to revise the offsite power supply requirements contained in Technical Specification 3.7.A.3. The proposed change also included several editorial changes including the correction of typographical errors in two other specifications on the same page and the capitalization of definitions where they appear in the specifications. Suggested changes to the Technical Specification bases were also provided. A mark-up of the proposed Technical Specification change on page 3.7-1 was contained in Enclosure 2 of the Reference. Proposed bases changes were also indicated on page 3.7-3.

The enclosure to this letter provides replacement pages 3.7-1, 3.7-3, 3.7-4 and 3.7-5 for the Oyster Creek Technical Specifications incorporating the changes contained on the marked-up pages previously submitted to the NRC in the Reference. Page 3.7-5 is a new page that was needed as a result of relocating the start of the bases to page 3.7-4 from 3.7-3. The amendment numbers on the bottom of pages 3.7-3, 3.7-4 and 3.7-5 containing the bases were revised as a result of the bases relocation to accurately reflect the text that the amendments apply to.

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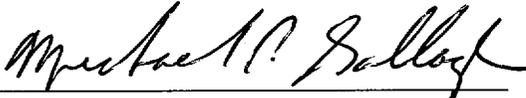
Oyster Creek Generating Station
2130-01-20259
Page 2 of 2

Should you have questions or require additional information please contact Mr. Paul F. Czaya at 610-765-5952.

I declare under penalty of perjury that the foregoing is true and correct.

Very truly yours,

12-20-01
Executed On



Michael P. Gallagher
Director, Licensing
Mid-Atlantic Regional Operating Group

Enclosure: Technical Specification Replacement Pages 3.7-1, 3.7-3, 3.7-4 and 3.7-5

c: H. J. Miller, Administrator, USNRC Region I
L. A. Dudes, USNRC Senior Resident Inspector, Oyster Creek
H. N. Pastis, USNRC Senior Project Manager, Oyster Creek
File No. 01037

Enclosure

Technical Specification Replacement Pages
3.7-1, 3.7-3, 3.7-4 and 3.7-5

3.7 AUXILIARY ELECTRICAL POWER

Applicability: Applies to the OPERATING status of the auxiliary electrical power supply.

Objective: To assure the OPERABILITY of the auxiliary electrical power supply.

Specifications:

A. The reactor shall not be made critical unless all of the following requirements are satisfied:

1. The following buses or panels energized.
 - a. 4160 volt buses 1C and 1D in the turbine building switchgear room.
 - b. 460 volt buses 1A2, 1B2, 1A21, 1B21 vital MCC 1A2 and 1B2 in the reactor building switchgear room: 1A3 and 1B3 at the intake structure; 1A21A, 1B21A, 1A21B, and 1B21B and vital MCC 1AB2 on 23'6" elevation in the reactor building; 1A24 and 1B24 at the stack.
 - c. 208/120 volt panels 3, 4, 4A, 4B, 4C and VACP-1 in the reactor building switchgear room.
 - d. 120 volt protection panel 1 and 2 in the cable room.
 - e. 125 volt DC distribution centers C and B, and panel D, Panel DC-F, isolation valve motor control center DC-1 and 125V DC motor control center DC-2.
 - f. 24 volt D.C. power panels A and B in the cable room.
2. One 230 KV line is fully operational and switch gear and both startup transformers are energized to carry power to the station 4160 volt AC buses and carry power to or away from the plant.
3. An additional source of power consisting of one of the following is in service connected to feed the appropriate plant 4160 V bus or buses:
 - a. A 69 KV line fully operational.
 - b. A 34.5 KV line fully operational.
4. Station batteries B and C and an associated battery charger are OPERABLE. Switchgear control power for 4160 volt bus 1D and 460 volt buses 1B2 and 1B3 are provided by battery B. Switchgear control power for 4160 volt bus 1C and 460 volt buses 1A2 and 1A3 are provided by battery C.
5. Bus tie breakers ED and EC are in the open position.

B. The reactor shall be PLACED IN the COLD SHUTDOWN CONDITION if the availability of power falls below that required by Specification A above, except that

1. The reactor may remain in operation for a period

-AND-

- 2) The reactor cavity shall be flooded above elevation 117 feet with the spent fuel pool gates removed, or all reactor fuel shall be contained in the spent fuel pool with spent fuel pool gates installed.

AND

- 3) The plant shall be placed in a configuration in which the core spray system is not required to be OPERABLE. |

Bases

The general objective is to assure an adequate supply of power with at least one active and one standby source of power available for operation of equipment required for a safe plant shutdown, to maintain the plant in a safe shutdown condition and to operate the required engineered safety feature equipment following an accident.

AC power for shutdown and operation of engineered safety feature equipment can be provided by any of three active (one or two 230 KV lines, one 69 KV line, and one 34.5 KV line) and either of two standby (two diesel generators) sources of power. (In applying the minimum requirement of one active and one standby source of AC power, since both 230 KV lines are on the same set of towers, either one or both 230 KV lines are considered as a single active source.) Normally all six sources are available. However, to provide for maintenance and repair of equipment and still have redundancy of power sources the requirement of one active and one standby source of power was established. The plant's main generator is not given credit as a source since it is not available during shutdown.

The plant 125V DC system consists of three batteries and associated distribution system. Batteries B and C are designated as the safety related subsystems while battery A is designated as a non-safety related subsystem. Safety related loads are supplied by batteries B and C, each with two associated full capacity chargers. One charger on each battery is in service at all times with the second charger available in the event of charger failure. These chargers are active sources and supply the normal 125V DC requirements with the batteries and standby sources. (1)

The probability analysis in Appendix "L" of the FDSAR was based on one diesel and shows that even with only one diesel the probability of requiring engineered safety features at the same time as the second diesel fails is quite small. The analysis used information on peaking diesels when synchronization was required which is not the case for Oyster Creek. Also the daily test of the second diesel when one is temporarily out of service tends to improve the reliability as does the fact that synchronization is not required.

As indicated in Amendment 18 to the Licensing Application, there are numerous sources of diesel fuel which can be obtained within 6 to 12 hours and the heating boiler fuel in a 75,000 gallon tank on the site could also be used. As indicated in Amendment 32 of the Licensing Application and including the Security System loads, the load requirement for the loss of offsite power would require 12,410 gallons for a three day supply. For the case of loss of offsite power plus loss-of-coolant plus bus failure 9790 gallons would be required for a three day supply.

In the case of loss of offsite power plus loss-of-coolant with both diesel generators starting the load requirements (all equipment operating) shown there would not be three days' supply. However, not all of this load is required for three days and, after evaluation of the conditions, loads not required on the diesel will be curtailed. It is reasonable to expect that within 8 hours conditions can be evaluated and the following loads curtailed:

1. One Core Spray Pump
2. One Core Spray Booster Pump
3. One Control Rod Drive Pump
4. One Containment Spray Pump
5. One Emergency Service Water Pump

With these pieces of equipment taken off at 8 hours after the incident it would require a total consumption of 12,840 gallons for a three day supply. Therefore, a minimum technical specification requirement of 14,000 gallons of diesel fuel in the standby diesel generator fuel tank will exceed the engineered safety features operational requirement after an accident by approximately 9%.

During plant cold shutdown or refueling, it may be necessary to inspect, repair and replace the 15,000 gallon standby diesel generator fuel storage tank. This would require tank partial or full drain down. An alternate fuel supply configuration may be established which consists of temporary tanker trucks capable of containing 14,000 gallons. This configuration is capable of supporting continuous operation of both diesels for at least 3 days.

The temporary configuration is acceptable since a minimal power load would be required during and following a design basis condition of a loss of offsite power while the plant is in cold shutdown or refueling. Analysis shows that in the event of a tornado or seismic event which may cause a loss of offsite power and a temporary loss of the temporary EDG fuel oil supply, power can be restored before the consequences of previously analyzed conditions are exceeded.

References:

- (1) Letter, Ivan R. Finfrock, Jr. to the Director of Nuclear Reactor Regulation dated April 4, 1978.