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Nuclear

RS-01-290

December 17, 2001

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555 - 0001

Quad Cities Nuclear Power Station, Units 1 and 2 Facility Operating License Nos. DPR-29 and DPR-30 NRC Docket Nos. 50-254 and 50-265

Subject:

Additional Information Supporting Technical Specification Changes to Reactor Pressure Vessel Level Instrumentation Surveillance Frequencies and Allowable Values Requirements

Reference:

Letter from Timothy J. Tulon, Exelon Generating Company (EGC), LLC, to U. S. NRC, "Request for Technical Specifications Changes to Modify Reactor Pressure Vessel Level Instrumentation Surveillance Frequencies and Allowable Values Requirements," dated August 13, 2001

In the above reference, Exelon Generation Company (EGC), LLC, submitted a Technical Specification amendment request for the Quad Cities Nuclear Power Station (QCNPS), Units 1 and 2. The proposed change would modify certain surveillance requirement frequencies and allowable values to reflect a planned upgrade to the reactor pressure vessel (RPV) water level instrumentation.

In a telephone conference call on November 29, 2001, Mr. A. R. Haeger (EGC) and Mr. S. N. Bailey (U.S. NRC) discussed a request for additional information regarding the proposed change. The attachment to this letter provides the requested information.

Par 1/23/02

December 17, 2001 U. S. Nuclear Regulatory Commission Page 2

If you have any questions concerning this letter, please contact D. L. Cecchett at (630) 657-2826.

Respectfully,

K. R. Jury

Director - Licensing

Midwest Regional Operating Group

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Attachment - Additional Information Supporting Technical Specification Changes to Reactor Pressure Vessel Level Instrumentation Surveillance Frequencies and Allowable Values Requirements

cc: Regional Administrator – NRC Region III

NRC Senior Resident Inspector – Quad Cities Nuclear Power Station

Office of Nuclear Facility Safety – Illinois Department of Nuclear Safety

STATE OF ILLINOIS)	
COUNTY OF DUPAGE)	
IN THE MATTER OF)	
EXELON GENERATION COMPANY, LLC)	Docket Numbers
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 A)	50-254 AND 50-265
SUBJECT: Supplemental Information Supporting Technical Specification change for Reactor Protection System Instrumentation Scram Discharge Volume Water Level - High				
<u>AFFIDAVIT</u>				
I affirm that the content of this transmittal is true and correct to the best of my knowledge, information and belief. Lighthamper				
	rn to before me, a Notary Publi named, this <u>' ' ' '</u> day '	of	W.	И '. 1

OFFICIAL SEAL
ANESE L. GRIGSBY
NOTARY PUBLIC, STATE OF ILLINOIS
MY COMMISSION EXPIRES 3-13-2005

Attachment

Supplemental Information Supporting Technical Specification change for Reactor Protection System Instrumentation Scram Discharge Volume Water Level - High

Question:

Describe the process used to determine the allowable value for the scram discharge volume level.

Response:

The Allowable Value (AV) for the scram discharge instrument volume has been calculated using Exelon Generation Company (EGC), LLC, methodology as defined in procedure NES-EIC-20.04, "Analysis of Instrument Channel Setpoint Error and Instrument Loop Accuracy." NES-EIC-20.04 has been reviewed and accepted by the NRC as part of the Quad Cities Nuclear Power Station (QCNPS) implementation of Improved Technical Specifications (NRC Letter from J. Mendiola to Exelon Generation Company, LLC, dated March 30, 2001). No changes to the Analytical Limit (AL) were made during the change to the new Magnetrol level switches. A QCNPS evaluation and placement of the new level switches allowed the AV to also remain unchanged. Differences in the time response characteristics of the Magnetrol compared to the FCI switches resulted in significant margin between the AV and AL as demonstrated in the evaluation.