

September 22, 1983

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Docket No. 50-324

Mr. E. E. Utley
Executive Vice President
Carolina Power & Light Company
P. O. Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment No. 81 to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant, Unit 2. The amendment consists of changes to the Technical Specifications in response to your application of June 16, 1982, as supplemented by letter dated July 22, 1983.

The amendment changes the Unit 2 Technical Specifications with regard to the instrumentation used to monitor the water level in the suppression chamber of the primary containment system (NUREG-0737 Item II.F.1.5).

The remainder of the items addressed in the June 16, 1982 application, including containment water level on Unit 1, will be the subject of a future action.

A copy of the Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Sam D. MacKay, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

- 1. Amendment No. 81 to DPR-62
- 2. Safety Evaluation

cc w/enclosures:
See next page

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[Handwritten signatures and initials]

OFFICE	ORB#2:DL	ORB#2:DL	ORB#2:DL	AD-OR:DL	OELD	
SURNAME	SNorris:km	SMacKay	DVassallo	GLainas	Concurred in by M. Harmon in memo	
DATE	9/13/83	9/13/83	9/14/83	9/14/83	9/15/83	

Mr. E. E. Utley
Carolina Power & Light Company
Brunswick Steam Electric Plant, Units 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 81
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company dated June 16, 1982 and July 22, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 81, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 22, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 81

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Remove the following pages and replace with identically numbered pages:

3/4 3-51 and 3/4 3-52

TABLE 3.3.5.3-1

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>MINIMUM NO. OF OPERABLE INSTRUMENT CHANNELS</u>
1. Reactor Vessel Pressure (B21-PI-R004A,B; C32-LPR-R608; and C32-PT-N005A,B)	2
2. Reactor Vessel Water Level (B21-LITS-N026A,B; B21-LR-R615; B21-LI-R604A,B; B21-LT-N037; and B21-LTM-N037-1)	2
3. Suppression Chamber Water Level (CAC-LT-2601; CAC-LI-2601-1) (CAC-LT-2602; CAC-LR-2602)	2
4. Suppression Chamber Water Temperature (CAC-TR-1258-14, 21; and C91-P602)	2
5. Suppression Chamber Atmosphere Temperature (CAC-TR-1258-17 thru 20; and C91-P602)	2
6. Drywell Pressure (CAC-PI-4176; CAC-PT-4176; CAC-PR-1257-1; and CAC-PT-4175)	2
7. Drywell Temperature (CAC-TR-1258-1 thru 13, 22, 23, 24; and C91-P602)	2
8. Drywell Radiation (CAC-AR-1260; CAC-AQH-1260-1, 2, 3; CAC-AR-1261; CAC-AQH-1261-1, 2, 3; CAC-AR-1262; and CAC-AQH-1262-1, 2, 3)	2
9. Drywell Oxygen (CAC-AT-1259-2; CAC-AR-1259; CAC-AT-1263-2; and CAC-AR-1263)	2
10. Drywell Hydrogen (CAC-AT-1259-1; CAC-AR-1259; CAC-AT-1263-1; and CAC-AR-1263)	2
11. Safety Relief Valve Position Indication: a. Primary - Sonic (B21-FY-4157 thru 4167) b. Secondary - Temp. (B21-TR-R614, points 1-11)	1/valve

TABLE 4.3.5.3-1

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure (B21-PI-RO04A,B; C32-LPR-R608; and C32-PT-NO05A,B)	M	R
2. Reactor Vessel Water Level (B21-LITS-NO26A,B; B21-LR-R615; B21-LI-R604A,B; B21-LT-NO37; and B21-LTM-NO37-1)	M	R
3. Suppression Chamber Water Level (CAC-LT-2601; CAC-LI-2601-1; CAC-LT-2602; CAC-LR-2602)	M	R
4. Suppression Chamber Water Temperature (CAC-TR-1258-14, 21; and C91-P602)	M	R
5. Suppression Chamber Atmosphere Temperature (CAC-TR-1258-17 thru 20; and C91-P602)	M	R
6. Drywell Pressure (CAC-PI-4176; CAC-PT-4176; CAC-PR-1257-1; and CAC-PT-4175)	M	R
7. Drywell Temperature (CAC-TR-1258-1 thru 13, 22, 23, 24; and C91-P602)	M	R
8. Drywell Radiation (CAC-AR-1260; CAC-AQH-1260-1,2,3; CAC-AR-1261; CAC-AQH-1261-1,2,3; CAC-AR-1262; and CAC-AQH-1262-1,2,3)	M	R
9. Drywell Oxygen Concentration (CAC-AT-1259-2; CAC-AR-1259; CAC-AT-1263-2; and CAC-AR-1263)	M	R
10. Drywell Hydrogen Concentration (CAC-AT-1259-1; CAC-AR-1259; CAC-AT-1263-1; and CAC-AR-1263)	M	R
11. Safety Relief Valve Position Indication		
a. Primary - Sonic (B21-FY-4157 thru 4167)	M	R
b. Secondary - Temp. (B21-TR-R614, points 1-11)	M	R



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 81 TO FACILITY OPERATING LICENSE NO. DPR-62

CAROLINA POWER & LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

DOCKET NO. 50-324

1.0 Introduction

By letter dated June 16, 1982 and July 22, 1983, the Carolina Power & Light Company (the licensee) submitted proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant (BSEP), Unit 2. The proposed changes would revise the Technical Specifications to reflect the installation of new instrumentation to measure the water level in the containment suppression chambers as recommended in Item II.F.1.5 of NUREG-0737, "Clarification of TMI Action Plan Requirements." On May 5, 1983, the NRC issued a Confirmatory Order that required the installation of this instrumentation by September 30, 1983. The new instrumentation was installed during plant operation and was fully operational on September 12, 1983. However, the licensee has requested the issuance of its proposed changes to the Technical Specifications before it declares the new instrumentation operable and, therefore, in conformance with the NRC Order and Technical specifications.

2.0 Discussion and Evaluation

NUREG-0737 Item II.F.1.5, concerns wide-range containment (suppression chamber) water level monitoring. The indication range for the Brunswick suppression pool was increased from (-6' to +6') to a new range of (-10' to +6') as a result of the requirements outlined in NUREG-0737.

The original configuration in which one transmitter drives both a narrow- and wide-range indicator was also changed. The original transmitter will continue to provide wide-range indication, and a new transmitter (CAC-LT-4177) was installed for narrow-range indication and annunciation. Only the wide-range indicator is required as part of the Post Accident Monitoring Instrumentation in the Technical Specifications. The proposed revisions to Tables 3.3.5.3-1 and 4.3.5.3-1 incorporate new instrument tag numbers due to the above mentioned modifications to the suppression chamber water level monitors.

We have reviewed the proposed Technical Specifications and find that they are administrative changes to reflect the installation of instrumentation as recommended in NUREG-0737. Therefore, we find the proposed Technical Specifications acceptable.

3.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Sam MacKay

Dated: September 22, 1983