

May 22, 1984

Docket No. 50-324

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Mr. E. E. Utley  
Executive Vice President  
Carolina Power & Light Company  
Post Office Box 1551  
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment No. 97 to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant, Unit 2. The amendment consists of changes to the Technical Specifications in response to your submittal of January 26, 1983. The January 26, 1983 submittal also applied to Unit 1. The same instrumentation was installed earlier in Unit 1 and the Technical Specifications were changed by Amendment No. 60 to Unit 1 dated December 12, 1983.

The amendment modifies the Technical Specifications to apply to new analog (continuous measuring) instrumentation that has been installed in Unit 2. The analog instrumentation replaces certain pressure switches and will provide improved performance of trip functions for reactor protection system actuation containment isolation, reactor core isolation cooling system isolation and emergency core cooling system actuation. In addition, miscellaneous typographical errors are corrected.

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 97 to License No. DPR-62
2. Safety Evaluation

cc w/enclosures:  
See next page

DL:ORB#2  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 97  
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power & Light Company (the licensee) dated January 26, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

8406040251 840522  
PDR ADOCK 05000324  
P PDR

Mr. E. E. Utley  
Carolina Power & Light Company  
Brunswick Steam Electric Plant, Units 1 and 2

cc:

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Carolina Power & Light Company  
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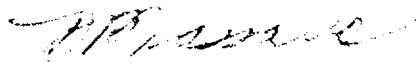
Dayne H. Brown, Chief  
Radiation Protection Branch  
Division of Facility Services  
Department of Human Resources  
Post Office Box 12200  
Raleigh, North Carolina 27605

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 97, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: May 22, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 97

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Revise the Appendix A Technical Specifications as indicated below. The changed areas are indicated by vertical lines.

<u>Remove</u>	<u>Insert</u>
2-4	2-4
3/4 3-3	3/4 3-3
3/4 3-6a	3/4 3-6a
3/4 3-8	3/4 3-8
3/4 3-11	3/4 3-11
3/4 3-12	3/4 3-12
3/4 3-13	3/4 3-13
3/4 3-14	3/4 3-14
3/4 3-14a	3/4 3-14a
3/4 3-15	3/4 3-15
3/4 3-17	3/4 3-17
3/4 3-18	3/4 3-18
3/4 3-19	3/4 3-19
3/4 3-20	3/4 3-20
3/4 3-21	3/4 3-21
3/4 3-22	3/4 3-22
3/4 3-23	3/4 3-23
3/4 3-24	3/4 3-24
3/4 3-24a	3/4 3-24a
3/4 3-25	3/4 3-25
3/4 3-26	3/4 3-26
3/4 3-27	3/4 3-27
3/4 3-28	3/4 3-28
3/4 3-29	3/4 3-29
3/4 3-29a	3/4 3-29a
-	3/4 3-29b
-	3/4 3-29c
3/4 3-31	3/4 3-31
3/4 3-32	3/4 3-32
3/4 3-32a	3/4 3-32a
3/4 3-34	3/4 3-34
3/4 3-35	3/4 3-35
3/4 3-35a	3/4 3-35a
3/4 3-37	3/4 3-37
3/4 3-38	3/4 3-38
3/4 3-38a	3/4 3-38a

TABLE 2.2.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. Intermediate Range Monitor, Neutron Flux - High <sup>(1)</sup> (C51-IRM-K601A,B,C,D,E,F,G,H)	$\leq$ 120 divisions of full scale	$\leq$ 120 divisions of full scale
2. Average Power Range Monitor (C51-APRM-CH.A,B,C,D,E,F)		
a. Neutron Flux - High, 15% <sup>(2)</sup>	$\leq$ 15% of RATED THERMAL POWER	$\leq$ 15% of RATED THERMAL POWER
b. Flow Biased Neutron Flux - High <sup>(3)(4)</sup>	$\leq$ (0.66 W + 54%)	$\leq$ (0.66 W + 54%)
c. Fixed Neutron Flux - High <sup>(4)</sup>	$\leq$ 120% of RATED THERMAL POWER	$\leq$ 120% of RATED THERMAL POWER
3. Reactor Vessel Steam Dome Pressure - High (B21-PTM-NO23A-1,B-1,C-1,D-1)	$\leq$ 1045 psig	$\leq$ 1045 psig
4. Reactor Vessel Water Level - Low, Level 1 (B21-LTM-NO17A-1,B-1,C-1,D-1)	$\geq$ +162.5 inches*	$\geq$ +162.5 inches*
5. Main Steam Line Isolation Valve - Closure <sup>(5)</sup> (B21-FO22A,B,C,D; B21-FO28A,B,C,D)	$\leq$ 10% closed	$\leq$ 10% closed
6. Main Steam Line Radiation - High (D12-RM-K603A,B,C,D)	$\leq$ 3 x full power background	$\leq$ 3.5 x full power background
7. Drywell Pressure - High (C72-PTM-NO02A-1,B-1,C-1,D-1)	$\leq$ 2 psig	$\leq$ 2 psig
8. Scram Discharge Volume Water Level - High (C12-LSH-NO13A,B,C,D) (C12-LSH-4516A,B,C,D)	$\leq$ 109 gallons	$\leq$ 109 gallons

TABLE 3.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM (a)</u>	<u>ACTION</u>
7. Drywell Pressure - High (C72-PT-NOO2A,B,C,D) (C72-PTM-NOO2A-1,B-1,C-1,D-1)	1, 2 <sup>(e)</sup>	2	6
8. Scram Discharge Volume Water Level - High (C12-LSH-NO13A,B,C,D) (C12-LSH-4516A,B,C,D)	1, 2, 5 <sup>(f)</sup>	2	5
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	1 <sup>(g)</sup>	4	8
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSL-1756,1757,1758,1759)	1 <sup>(g)</sup>	4	8
11. Reactor Mode Switch in Shutdown Position (C72A-S1)	1, 2, 3, 4, 5	1	9
12. Manual Scram (C72A-S3A,B)	1, 2, 3, 4, 5	1	10

TABLE 3.3.1-2 (Continued)

REACTOR PROTECTION SYSTEM RESPONSE TIMES

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>RESPONSE TIME (Seconds)</u>
7. Drywell Pressure - High (C72-PT-N002A,B,C,D) (C72-PTM-N002A-1,B-1,C-1,D-1)	NA
8. Scram Discharge Volume Water Level - High (C12-LSH-N013A,B,C,D) (C12-LSH-4516A,B,C,D)	NA
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	$\leq 0.06$
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSL-1756,1757,1758,1759)	$\leq 0.08$
11. Reactor Mode Switch in Shutdown Position (C72A-S1)	NA
12. Manual Scram (C72A-S3A,B)	NA

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\* Neutron detectors are exempt from response time testing. Response time shall be measured from detector output or from the input of the first electronic component in the channel.

TABLE 4.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION<sup>(a)</sup></u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
5. Main Steam Line Isolation Valve - Closure (B21-F022A,B,C,D and B21-F028A,B,C,D)	NA	M	R <sup>(h)</sup>	1
6. Main Steam Line Radiation - High (D12-RM-K603A,B,C,D)	S	M <sup>(i)</sup>	R <sup>(j)</sup>	1, 2
7. Drywell Pressure - High (C72-PT-N002A,B,C,D) (C72-PTM-N002A-1,B-1,C-1,D-1)	NA <sup>(k)</sup>	NA	R <sup>(l)</sup>	1, 2
	D	M	M	1, 2
8. Scram Discharge Volume Water Level - High (C12-LSH-N013A,B,C,D) (C12-LSH-4516A,B,C,D)	NA	Q	R	1, 2, 5
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	NA	M	R <sup>(h)</sup>	1
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSL-1756,1757,1758,1759)	NA	M	R	1
11. Reactor Mode Switch in Shutdown Position (C72A-S1)	NA	R	NA	1, 2, 3, 4, 5
12. Manual Scram (C72A-S3A,B)	NA	Q	NA	1, 2, 3, 4, 5

TABLE 3.3.2-1

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
<u>1. PRIMARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level -				
1. Low, Level 1 (B21-LT-N017A-1,B-1,C-1,D-1) (B21-LTM-N017A-1,B-1,C-1,D-1)	2, 6, 7, 8	2	1, 2, 3	20
2. Low, Level 2 (B21-LT-N024A-1,B-1, and B21-LT-N025A-1,B-1)  (B21-LTM-N024A-1,B-1 and B21-LTM-N025A-1,B-1)	1, 3	2	1, 2, 3	20
b. Drywell Pressure - High (C72-PT-N002A,B,C,D) (C72-PTM-N002A-1,B-1,C-1,D-1)	2, 6, 7	2	1, 2, 3	20
c. Main Steam Line				
1. Radiation - High (d) (D12-RM-K603A,B,C,D)	1	2	1, 2, 3	21
2. Pressure - Low (B21-PT-N015A,B,C,D) (B21-PTM-N015A-1,B-1,C-1,D-1)	1	2	1	22
3. Flow - High (B21-PDT-N006A,B,C,D; B21-PDT-N007A,B,C,D; B21-PDT-N008A,B,C,D; B21-PDT-N009A,B,C,D)  (B21-PDTM-N006A-1,B-1,C-1,D-1; B21-PDTM-N007A-1,B-1,C-1,D-1; B21-PDTM-N008A-1,B-1,C-1,D-1; B21-PDTM-N009A-1,B-1,C-1,D-1)	1	2/line	1	22

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>				
4. Flow - High (B21-PDTS-N006A-2; B21-PDTS-N007B-2; B21-PDTS-N008C-2; B21-PDTS-N009D-2)	1	2	2, 3	21
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A,B,C,D; B21-TS-N011A,B,C,D; B21-TS-N012A,B,C,D; B21-TS-N013A,B,C,D)	1	2 <sup>(e)</sup>	1, 2, 3	21
e. Condenser Vacuum - Low (B21-PT-N056A,B,C,D) (B21-PTM-N056A-1,B-1,C-1,D-1)	1	2	1, 2 <sup>(f)</sup>	21
f. Turbine Building Area Temperature - High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	1	4 <sup>(e)</sup>	1, 2, 3	21

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TABLE 3.3.2-1 (Continued)

<u>ISOLATION ACTUATION INSTRUMENTATION</u>				
<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
<u>2. SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Building Exhaust Radiation - High (D12-RM-N010A,B)	6	1	1, 2, 3, 5, and*	23
b. Drywell Pressure - High (C72-PT-N002A,B,C,D) (C72-PTM-N002A-1,B-1,C-1,D-1)	2, 6, 7	2	1, 2, 3	23
c. Reactor Vessel Water Level - Low, Level 2 (B21-LT-N024A-1,B-1 and B21-LT-N025A-1,B-1) (B21-LTM-N024A-1,B-1 and B21-LTM-N025A-1,B-1)	1, 3	2	1, 2, 3	23
<u>3. REACTOR WATER CLEANUP SYSTEM ISOLATION</u>				
a. $\Delta$ Flow - High (G31-dFS-N603-1A,1B)	3	1	1, 2, 3	24
b. Area Temperature - High (G31-TS-N600A,B,C,D,E,F)	3	2	1, 2, 3	24
c. Area Ventilation $\Delta$ Temp. - High (G31-TS-N602A,B,C,D,E,F)	3	2	1, 2, 3	24
d. SLCS Initiation (C41A-S1)	3 (g)	NA	1, 2, 3	24
e. Reactor Vessel Water Level - Low, Level 2 (B21-LT-N024 A-1,B-1 and B21-LT-N025 A-1,B-1) (B21-LTM-N024 A-1,B-1 and B21-LTM-N025 A-1,B-1)	1, 3	2	1, 2, 3	24

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
<b>4. CORE STANDBY COOLING SYSTEMS ISOLATION</b>				
<b>a. High Pressure Coolant Injection System Isolation</b>				
1. HPCI Steam Line Flow - High (E41-PDT-N004; E41-PDT-N005)  (E41-PDTS-N004-2; E41-PDTS-N005-2)	4	1	1, 2, 3	25
2. HPCI Steam Line High Flow Time Delay Relay (E41-TDR-K33; E41-TDR-K43)	NA	1	1, 2, 3	25
3. HPCI Steam Supply Pressure - Low (E41-PSL-N001A,B,C,D)	4	2	1, 2, 3	25
4. HPCI Steam Line Tunnel Temperature - High (E41-TS-3314; E41-TS-3315; E41-TS-3316; E41-TS-3317; E41-TS-3318; E41-TS-3354; E41-TS-3488; E41-TS-3489)	4	2	1, 2, 3	25
5. Bus Power Monitor (E41-K55 and E41-K56)	NA <sup>(h)</sup>	1/bus	1, 2, 3	26
6. HPCI Turbine Exhaust Diaphragm Pressure - High (E41-PSH-N012A,B,C,D)	4	2	1, 2, 3	25

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
7. HPCI Steam Line Ambient Temperature - High (E51-TS-N603C,D)	4	1	1, 2, 3	25
8. HPCI Steam Line Area $\Delta$ Temp. - High (E51-dTS-N604C,D)	4	1	1, 2, 3	25
9. Emergency Area Cooler Temperature - High (E41-TS-N602A,B)	4	1	1, 2, 3	25
b. Reactor Core Isolation Cooling System Isolation				
1. RCIC Steam Line Flow - High (E51-PDT-N017; E51-PDT-N018)  (E51-PDTS-N017-2; E51-PDTS-N018-2)	5	1	1, 2, 3	25
2. RCIC Steam Line High Flow Time Delay Relay (E51-TDR-K32; E51-TDR-K12)	NA	1	1, 2, 3	25
3. RCIC Steam Supply Pressure - Low (E51-PS-N019A,B,C,D)	5	2	1, 2, 3	25
4. RCIC Steam Line Tunnel Temperature - High (E51-TS-3319; E51-TS-3320; E51-TS-3321; E51-TS-3322; E51-TS-3323; E51-TS-3355; E51-TS-3487)	5	2	1, 2, 3	25

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
5. Bus Power Monitor (E51-K42 and E51-K43)	NA (h)	1/bus	1, 2, 3	26
6. RCIC Turbine Exhaust Diaphragm Pressure - High (E51-PS-N012A,B,C,D)	5	2	1, 2, 3	25
7. RCIC Steam Line Ambient Temp - High (E51-TS-N603A,B)	5	1	1, 2, 3	25
8. RCIC Steam Line Area $\Delta$ Temp - High (E51-dTS-N604A,B)	5	1	1, 2, 3	25
9. RCIC Equipment Room Ambient Temp - High (E51-TS-N602A,B)	5	1	1, 2, 3	25
10. RCIC Equipment Room $\Delta$ Temp - High (E51-dTS-N601A,B)	5	1	1, 2, 3	25
<u>5. SHUTDOWN COOLING SYSTEM ISOLATION</u>				
a. Reactor Vessel Water Level - Low, Level 1 (B21-LT-N017A-1,B-1,C-1,D-1) (B21-LTM-N017A-1,B-1,C-1,D-1)	2, 6, 7, 8	2	1, 2, 3	27
b. Reactor Steam Dome Pressure-High (B32-PS-N018A,B)	7, 8	1	1, 2, 3	27

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>1. PRIMARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level -		
1. Low, Level 1 (B21-LTM-N017A-1,B-1,C-1,D-1)	$\geq + 162.5$ inches*	$\geq + 162.5$ inches*
2. Low, Level 2 (B21-LTM-N024A-1,B-1 and B21-LTM-N025A-1,B-1)	$\geq + 112$ inches*	$\geq + 112$ inches*
b. Drywell Pressure - High (C72-PTM-N002A-1,B-1,C-1,D-1)	$\leq 2$ psig	$\leq 2$ psig
c. Main Steam Line		
1. Radiation - High (D12-RM-K603A,B,C,D)	$\leq 3$ x full power background	$\leq 3.5$ x full power background
2. Pressure - Low (B21-PTM-N015A-1,B-1,C-1,D-1)	$\geq 825$ psig	$\geq 825$ psig
3. Flow - High (B21-PDTM-N006A-1,B-1,C-1,D-1; B21-PDTM-N007A-1,B-1,C-1,D-1; B21-PDTM-N008A-1,B-1,C-1,D-1; B21-PDTM-N009A-1,B-1,C-1,D-1)	$\leq 140\%$ of rated flow	$\leq 140\%$ of rated flow
4. Flow - High (B21-PDTS-N006A-2; B21-PDTS-N007B-2; B21-PDTS-N008C-2; B21-PDTS-N009D-2)	$\leq 40\%$ of rated flow	$\leq 40\%$ of rated flow

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>		
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A,B,C,D; B21-TS-N011A,B,C,D; B21-TS N012A,B,C,D; B21-TS-N013A,B,C,D)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
e. Condenser Vacuum - Low (B21-PTM-N056A-1,B-1,C-1,D-1)	$\geq 7$ inches Hg vacuum	$\geq 7$ inches Hg vacuum
f. Turbine Building Area Temp - High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
<u>2. SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Building Exhaust Radiation - High (D12-RM-N010A,B)	$\leq 11$ mr/hr	$\leq 11$ mr/hr
b. Drywell Pressure - High (C72-PTM-N002A-1,B-1,C-1,D-1)	$\leq 2$ psig	$\leq 2$ psig
c. Reactor Vessel Water Level - Low, Level 2 (B21-LTM-N024A-1,B-1 and B21-LTM-N025A-1,B-1)	$\geq + 112$ inches*	$\geq + 112$ inches*

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<b>3. REACTOR WATER CLEANUP SYSTEM ISOLATION</b>		
a. $\Delta$ Flow - High (G31-dFS-N603-1A, 1B)	$\leq$ 53 gal/min	$\leq$ 53 gal/min
b. Area Temperature - High (G31-TS-N600A, B, C, D, E, F)	$\leq$ 150°F	$\leq$ 150°F
c. Area Ventilation Temperature $\Delta$ Temp - High (G31-TS-N602A, B, C, D, E, F)	$\leq$ 50°F	$\leq$ 50°F
d. SLCS Initiation (C41A-S1)	NA	NA
e. Reactor Vessel Water Level - Low, Level 2 (B21-LTM-NO24A-1, B-1 and B21-LTM-NO25A-1, B-1)	$\geq$ + 112 inches*	$\geq$ + 112 inches*
<b>4. CORE STANDBY COOLING SYSTEMS ISOLATION</b>		
a. High Pressure Coolant Injection System Isolation		
1. HPCI Steam Line Flow - High (E41-PDTS-NO04-2; E41-PDTS-NO05-2)	$\leq$ 300% of rated flow	$\leq$ 300% of rated flow
2. HPCI Steam Line High Flow Time Delay Relay (E41-TDR-K33; E41-TDR-K43)	$3 \leq t \leq 7$ seconds	$3 \leq t \leq 12$ seconds
3. HPCI Steam Supply Pressure - Low (E41-PSL-NO01A, B, C, D)	$\geq$ 100 psig	$\geq$ 100 psig

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
4. HPCI Steam Line Tunnel Temperature - High (E41-TS-3314; E41-TS-3315; E41-TS-3316; E41-TS-3317; E41-TS-3318; E41-TS-3354; E41-TS-3488; E41-TS-3489)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
5. Bus Power Monitor (E41-K55 and E41-K56)	NA	NA
6. HPCI Turbine Exhaust Diaphragm Pressure - High (E41-PSH-N012A,B,C,D)	$\leq 10$ psig	$\leq 10$ psig
7. HPCI Steam Line Ambient Temp - High (E51-TS-N603C,D)	$\leq 200^{\circ}\text{F}$	$\leq 200^{\circ}\text{F}$
8. HPCI Steam Line Area $\Delta$ Temp - High (E51-dTS-N604C,D)	$\leq 50^{\circ}\text{F}$	$\leq 50^{\circ}\text{F}$
9. Emergency Area Cooler Temp - High (E41-TS-N602A,B)	$\leq 175^{\circ}\text{F}$	$\leq 175^{\circ}\text{F}$
b. Reactor Core Isolation Cooling System Isolation		
1. RCIC Steam Line Flow - High (E51-PDTS-N017-2; E51-PDTS-N018-2)	$\leq 300\%$ of rated flow	$\leq 300\%$ of rated flow
2. RCIC Steam Line High Flow Time Delay Relay (E51-TDR-K32; E51-TDR-K12)	$3 \leq t \leq 7$ seconds	$3 \leq t \leq 12$ seconds

TABLE 3.3.2-2 (Continued)ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
3. RCIC Steam Supply Pressure - Low (E51-PS-N019A,B,C,D)	$\geq$ 50 psig	$\geq$ 50 psig
4. RCIC Steam Line Tunnel Temp - High (E51-TS-3319; E51-TS-3320; E51-TS-3321; E51-TS-3322; E51-TS-3323; E51-TS-3355; E51-TS-3487)	$\leq$ 175°F	$\leq$ 175°F
5. Bus Power Monitor (E51-K42 and E51-K43)	NA	NA
6. RCIC Turbine Exhaust Diaphragm Pressure - High (E51-PS-N012A,B,C,D)	$\leq$ 10 psig	$\leq$ 10 psig
7. RCIC Steam Line Ambient Temp - High (E51-TS-N603A,B)	$\leq$ 200°F	$\leq$ 200°F
8. RCIC Steam Line Area $\Delta$ Temp - High (E51-dTS-N604A,B)	$\leq$ 50°F	$\leq$ 50°F
9. RCIC Equipment Room Ambient Temp - High (E51-TS-N602A,B)	$\leq$ 175°F	$\leq$ 175°F
10. RCIC Equipment Room $\Delta$ Temp - High (E51-dTS-N601A,B)	$\leq$ 50°F	$\leq$ 50°F
5. <u>SHUTDOWN COOLING SYSTEM ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Level 1 (B21-LTM-N017A-1,B-1,C-1,D-1)	$\geq$ + 162.5 inches*	$\geq$ + 162.5 inches*
b. Reactor Steam Dome Pressure - High (B32-PS-N018A,B)	$\leq$ 140 psig	$\leq$ 140 psig

\*Vessel water levels refer to REFERENCE LEVEL ZERO.

TABLE 3.3.2-3ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>RESPONSE TIME (Seconds)#</u>
1. <u>PRIMARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level -	
1. Low, Level 1	$\leq 13$
(B21-LT-NO17A-1,B-1,C-1,D-1)	
(B21-LTM-NO17A-1,B-1,C-1,D-1)	
2. Low, Level 2	$\leq 1.0^*$
(B21-LT-NO24A-1,B-1 and	
B21-LT-NO25A-1,B-1)	
(B21-LTM-NO24A-1,B-1 and	
B21-LTM-NO25A-1,B-1)	
b. Drywell Pressure - High	$\leq 13$
(C72-PT-NO02A,B,C,D)	
(C72-PTM-NO02A-1,B-1,C-1,D-1)	
c. Main Steam Line	$\leq 1.0^*$
1. Radiation - High <sup>(b)</sup>	
(D12-RM-K603A,B,C,D)	
2. Pressure - Low	$\leq 13$
(B21-PT-NO15A,B,C,D)	
(B21-PTM-NO15A-1,B-1,C-1,D-1)	
3. Flow - High	$\leq 0.5^*$
(B21-PDT-NO06A,B,C,D;	
B21-PDT-NO07A,B,C,D;	
B21-PDT-NO08A,B,C,D;	
B21-PDT-NO09A,B,C,D)	
(B21-PDTM-NO06A-1,B-1,C-1,D-1;	
B21-PDTM-NO07A-1,B-1,C-1,D-1;	
B21-PDTM-NO08A-1,B-1,C-1,D-1;	
B21-PDTM-NO09A-1,B-1,C-1,D-1)	
4. Flow - High	$\leq 0.5^*$
(B21-PDTS-NO06A-2;	
B21-PDTS-NO07B-2;	
B21-PDTS-NO08C-2;	
B21-PDTS-NO09D-2)	
d. Main Steam Line Tunnel Temperature - High	$\leq 13$
(B21-TS-NO10A,B,C,D;	
B21-TS-NO11A,B,C,D;	
B21-TS-NO12A,B,C,D;	
B21-TS-NO13A,B,C,D)	
e. Condenser Vacuum - Low	$\leq 13$
(B21-PT-NO56A,B,C,D)	
(B21-PTM-NO56A-1,B-1,C-1,D-1)	

TABLE 3.3.2-3 (Continued)

TRIP FUNCTION AND INSTRUMENT NUMBER	RESPONSE TIME (Seconds)#
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>	
f. Turbine Building Area Temperature - High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	NA
2. <u>SECONDARY CONTAINMENT ISOLATION</u>	
a. Reactor Building Exhaust Radiation - High <sup>(b)</sup> (D12-RM-NO10A,B)	<13
b. Drywell Pressure - High (C72-PT-NO02A,B,C,D) (C72-PTM-NO02A-1,B-1,C-1,D-1)	<13
c. Reactor Vessel Water Level - Low, Level 2 (B21-LT-NO24A-1,B-1 and B21-LT-NO25A-1,B-1)  (B21-LTM-NO24A-1,B-1 and B21-LTM-NO25A-1,B-1)	<1.0*
3. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>	
a. $\Delta$ Flow - High (G31-dFS-N603-1A,1B)	<13
b. Area Temperature - High (G31-TS-N600A,B,C,D,E,F)	<13
c. Area Ventilation Temperature $\Delta T$ - High (G31-TS-N602A,B,C,D,E,F)	<13
d. SLCS Initiation (C41A-S1)	NA
e. Reactor Vessel Water Level - Low, Level 2 (B21-LT-NO24 A-1,B-1 and B21-LT-NO25 A-1,B-1)  (B21-LTM-NO24 A-1,B-1 and B21-LTM-NO25 A-1,B-1)	<1.0*

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>RESPONSE TIME (Seconds)#</u>
<u>4. CORE STANDBY COOLING SYSTEMS ISOLATION</u>	
a. High Pressure Coolant Injection System Isolation	
1. HPCI Steam Line Flow - High (E41-PDT-N004; E41-PDT-N005)  (E41-PDTS-N004-2; E41-PDTS-N005-2)	≤13 <sup>(a)##</sup>
2. HPCI Steam Line High Flow Time Delay Relay (E41-TDR-K33; E41-TDR-K43)	NA
3. HPCI Steam Supply Pressure - Low (E41-PSL-N001A,B,C,D)	≤13
4. HPCI Steam Line Tunnel Temperature - High (E41-TS-3314; E41-TS-3315; E41-TS-3316; E41-TS-3317; E41-TS-3318; E41-TS-3354; E41-TS-3488; E41-TS-3489)	≤13
5. Bus Power Monitor (E41-K55 and E41-K56)	NA
6. HPCI Turbine Exhaust Diaphragm Pressure - High (E41-PSH-N012A,B,C,D)	NA
7. HPCI Steam Line Ambient Temperature - High (E51-TS-N603C,D)	NA
8. HPCI Steam Line Area (E51-dTS-N604C,D)	NA
9. Emergency Area Cooler Temperature - High (E41-TS-602A,B)	NA

TABLE 3.3.2-3 (Continued)ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>RESPONSE TIME (Seconds)#</u>
b. Reactor Core Isolation Cooling System Isolation	
1. RCIC Steam Line Flow - High (E51-PDT-N017; E51-PDT-N018)  (E51-PDTS-N017-2; E51-PDTS-N018-2)	NA
2. RCIC Steam Line High Flow Time Delay Relay (E51-TDR-K32; E51-TDR-K12)	NA
3. RCIC Steam Supply Pressure - Low (E51-PS-N019A,B,C,D)	NA
4. RCIC Steam Line Tunnel Temp - High (E51-TS-3319; E51-TS-3320; E51-TS-3321; E51-TS-3322; E51-TS-3323; E51-TS-3355; E51-TS-3487)	NA
5. Bus Power Monitor (E51-K42 and E51-K43)	NA
6. RCIC Turbine Exhaust Diaphragm Pressure - High (E51-PS-N012A,B,C,D)	NA
7. RCIC Steam Line Ambient Temperature - High (E51-TS-N603A,B)	NA
8. RCIC Steam Line Area $\Delta$ Temp - High (E51-dTS-N604A,B)	NA
9. Emergency Area Cooler Temperature - High (E51-TS-N602A,B)	NA
10. RCIC Equipment Room $\Delta$ Temp - High (E51-dTS-N601A,B)	NA
5. <u>SHUTDOWN COOLING SYSTEM ISOLATION</u>	
a. Reactor Vessel Water Level - Low, Level 1 (B21-LT-N017A-1,B-1,C-1,D-1) (B21-LTM-N017A-1,B-1,C-1,D-1)	NA
b. Reactor Steam Dome Pressure - High (B32-PS-N018A,B)	NA

BRINSMICK - INTT 2

TABLE 4.3.2-1

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<b>1. PRIMARY CONTAINMENT ISOLATION</b>				
<b>a. Reactor Vessel Water Level -</b>				
1. Low, Level 1				
(B21-LT-NO17A-1,B-1,C-1,D-1)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
(B21-LTM-NO17A-1,B-1,C-1,D-1)	D	M	M	1, 2, 3
2. Low, Level 2				
(B21-LT-NO24A-1,B-1 and B21-LT-NO25A-1,B-1)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
(B21-LTM-NO24A-1,B-1 and B21-LTM-NO25A-1,B-1)	D	M	M	1, 2, 3
<b>b. Drywell Pressure - High</b>				
(C72-PT-NO02A,B,C,D)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
(C72-PTM-NO02A-1,B-1,C-1,D-1)	D	M	M	1, 2, 3
<b>c. Main Steam Line</b>				
1. Radiation - High				
(D12-RM-K603A,B,C,D)	D	W	R	1, 2, 3
2. Pressure - Low				
(B21-PT-NO15A,B,C,D)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1
(B21-PTM-NO15A-1,B-1,C-1,D-1)	D	M	M	1
3. Flow - High				
(B21-PDT-NO06A,B,C,D; B21-PDT-NO07A,B,C,D; B21-PDT-NO08A,B,C,D; B21-PDT-NO09A,B,C,D)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1
(B21-PDTM-NO06A-1,B-1,C-1,D-1; B21-PDTM-NO07A-1,B-1,C-1,D-1; B21-PDTM-NO08A-1,B-1,C-1,D-1; B21-PDTM-NO09A-1,B-1,C-1,D-1)	D	M	M	1

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<u>PRIMARY CONTAINMENT ISOLATION (Continued)</u>				
4. Flow - High (B21-PDTS-N006A-2; B21-PDTS-N007B-2; B21-PDTS-N008C-2; B21-PDTS-N009D-2)	D	M	M	2, 3
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010A,B,C,D; B21-TS-N011A,B,C,D; B21-TS-N012A,B,C,D; B21-TS-N013A,B,C,D)	NA	M	R	1, 2, 3
e. Condenser Vacuum - Low (B21-PT-N056A,B,C,D) (B21-PTM-N056A-1,B-1,C-1,D-1)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2# 1, 2#
f. Turbine Building Area Temp-High (B21-TS-3225A,B,C,D; B21-TS-3226A,B,C,D; B21-TS-3227A,B,C,D; B21-TS-3228A,B,C,D; B21-TS-3229A,B,C,D; B21-TS-3230A,B,C,D; B21-TS-3231A,B,C,D; B21-TS-3232A,B,C,D)	NA	M	R	1, 2, 3

BRUNSWICK - INTT 2

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<b>2. <u>SECONDARY CONTAINMENT ISOLATION</u></b>				
a. Reactor Building Exhaust Radiation - High (D12-RM-N010A,B)	D	M	R	1,2,3,5, and *
b. Drywell Pressure - High (C72-PT-N002A,B,C,D) (C72-PTM-N002A-1,B-1,C-1,D-1)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
	D	M	M	1, 2, 3
c. Reactor Vessel Water Level - Low, Level 2 (B21-LT-N024 A-1,B-1 and B21-LT-N025 A-1,B-1)  (B21-LTM-N024 A-1,B-1 and B21-LTM-N025 A-1,B-1)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
	D	M	M	1, 2, 3
<b>3. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u></b>				
a. $\Delta$ Flow - High (G31-dFS-N603-1A,1B)	D	M	R	1, 2, 3
b. Area Temperature - High (G31-TS-N600A,B,C,D,E,F)	NA	M	R	1, 2, 3
c. Area Ventilation $\Delta$ Temp - High (G31-TS-N602A,B,C,D,E,F)	NA	M	R	1, 2, 3
d. SLCS Initiation (C41A-S1)	NA	R	NA	1, 2, 3
e. Reactor Vessel Water Level - Low, Level 2 (B21-LT-N024 A-1,B-1 and B21-LT-N025 A-1,B-1)  (B21-LTM-N024 A-1,B-1 and B21-LTM-N025 A-1,B-1)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
	D	M	M	1, 2, 3

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<b>4. CORE STANDBY COOLING SYSTEMS ISOLATION</b>				
<b>a. High Pressure Coolant Injection System Isolation</b>				
1. HPCI Steam Line Flow - High (E41-PDT-N004; E41-PDT-N005)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
(E41-PDTS-N004-2; E41-PDTS-N005-2)	D	M	M	1, 2, 3
2. HPCI Steam Line High Flow Time Delay Relay (E41-TDR-K33; E41-TDR-K43)	NA	R	R	1, 2, 3
3. HPCI Steam Supply Pressure - Low (E41-PSL-N001A,B,C,D)	NA	M	R	1, 2, 3
4. HPCI Steam Line Tunnel Temperature - High (E41-TS-3314; E41-TS-3315; E41-TS-3316; E41-TS-3317; E41-TS-3318; E41-TS-3354; E41-TS-3388; E41-TS-3489)	NA	M	Q	1, 2, 3
5. Bus Power Monitor (E41-K55 and E41-K56)	NA	R	NA	1, 2, 3

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
6. HPCI Turbine Exhaust Diaphragm Pressure - High (E41-PSH-N012A,B,C,D)	NA	M	Q	1, 2, 3
7. HPCI Steam Line Ambient Temp - High (E51-TS-N603C,D)	NA	M	R	1, 2, 3
8. HPCI Steam Line Area $\Delta$ Temp - High (E51-dTS-N604C,D)	NA	M	R	1, 2, 3
9. Emergency Area Cooler Temp - High (E41-TS-N602A,B)	NA	M	Q	1, 2, 3
b. Reactor Core Isolation Cooling System Isolation				
1. RCIC Steam Line Flow - High (E51-PDT-N017; E51-PDT-N018)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
(E51-PDTM-N017-2; E51-PDTM-N018-2)	D	M	M	1, 2, 3
2. RCIC Steam Line High Flow Time Delay Relay (E51-TDR-K32; E51-TDR-K12)				
3. RCIC Steam Supply Pressure - Low (E51-PS-N019A,B,C,D)	NA	M	Q	1, 2, 3

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
4. RCIC Steam Line Tunnel High Temperature (E51-TS-3319; E51-TS-3320; E51-TS-3321; E51-TS-3322; E51-TS-3323; E51-TS-3355; E51-TS-3487)	NA	M	R	1, 2, 3
5. Bus Power Monitor (E51-K42 and E51-K43)	NA	R	NA	1, 2, 3
6. RCIC Turbine Exhaust Diaphragm Pressure - High (E51-PS-N012A,B,C,D)	NA	M	R	1, 2, 3
7. RCIC Steam Line Ambient Temp - High (E51-TS-N603A,B)	NA	M	R	1, 2, 3
8. RCIC Steam Line Area $\Delta$ Temp - High (E51-dTS-N604A,B)	NA	M	R	1, 2, 3
9. RCIC Equipment Room Ambient Temp - High (E51-TS-N602A,B)	NA	M	Q	1, 2, 3
10. RCIC Equipment Room $\Delta$ Temp - High (E51-dTS-N601A,B)	NA	M	Q	1, 2, 3

TABLE 4.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<b>5. <u>SHUTDOWN COOLING SYSTEM ISOLATION</u></b>				
a. Reactor Vessel Water Level - Low, Level 1 (B21-LT-N017A-1,B-1,C-1,D-1) (B21-LTM-N017A-1,B-1,C-1,D-1)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3 1, 2, 3
b. Reactor Steam Dome Pressure - High (B32-PS-N018A,B)	NA	S/U <sup>(c)</sup> , M	R	1, 2, 3

TABLE 4.3.2-1 (Continued)ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTSNOTES

- (a) The transmitter channel check is satisfied by the trip unit channel check. A separate transmitter check is not required.
- (b) Transmitters are exempted from the monthly channel calibration.
- (c) If not performed within the previous 31 days.
- \* When handling irradiated fuel in the secondary containment.
- # When reactor steam pressure  $\geq$  500 psig.

TABLE 3.3.3-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
<u>1. CORE SPRAY SYSTEM</u>			
a. Reactor Vessel Water Level - Low, Level 3 (B21-LT-NO31A,B,C,D) (B21-LTS-NO31A-4,B-4,C-4,D-4)	2	1, 2, 3, 4, 5	30
b. Reactor Steam Dome Pressure - Low (Injection Permissive) (B21-PT-NO21A,B,C,D) (B21-PTS-NO21A-2,B-2,C-2,D-2)	2	1, 2, 3, 4, 5	31
c. Drywell Pressure - High (E11-PT-NO11A,B,C,D) (E11-PTS-NO11A-2,B-2,C-2,D-2)	2	1, 2, 3	30
d. Time Delay Relay (E21-K16A,B)	1	1, 2, 3, 4, 5	31
e. Bus Power Monitor# (E21-K1A,B)	1/bus	1, 2, 3, 4, 5	32
<u>2. LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM</u>			
a. Drywell Pressure - High (E11-PT-NO11A,B,C,D) (E11-PTM-NO11A-1,B-1,C-1,D-1)	2	1, 2, 3	30
b. Reactor Vessel Water Level - Low, Level 3 (B21-LT-NO31A,B,C,D) (B21-LTS-NO31A-4,B-4,C-4,D-4)	2	1, 2, 3, 4*, 5*	30
c. Reactor Vessel Shroud Level (Drywell Spray Permissive) (B21-LT-NO36 and B21-LT-NO37) (B21-LTM-NO36-1 and B21-LTM-NO37-1)	1	1, 2, 3, 4*, 5*	31
d. Reactor Steam Dome Pressure - Low (Injection Permissive) (B21-PT-NO21A,B,C,D) (B21-PTM-NO21A-1,B-1,C-1,D-1) (B21-PTS-NO21A-2,B-2,C-2,D-2)			
1. RHR Pump Start and LPCI Injection Valve Actuation	2	1, 2, 3, 4*, 5*	31
2. Recirculation Loop Pump Discharge Valve Actuation	2	1, 2, 3, 4*, 5*	31

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
<u>LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM (Continued)</u>			
e. RHR Pump Start - Time Delay Relay (STR-2A1,2 and STR-2B1,2)	1	1, 2, 3, 4*, 5*	31
f. Bus Power Monitor# (E11-K106A,B)	1/bus	1, 2, 3, 4*, 5*	32
<u>3. HIGH PRESSURE COOLANT INJECTION SYSTEM</u>			
a. Reactor Vessel Water Level - Low, Level 2 (B21-LT-NO31A,B,C,D) (B21-LTS-NO31A-2,B-2,C-2,D-2)	2	1, 2, 3	30
b. Drywell Pressure - High (E11-PT-NO11A,B,C,D) (E11-PTS-NO11A-2,B-2,C-2,D-2)	2	1, 2, 3	30
c. Condensate Storage Tank Level - Low (E41-LS-NO02, E41-LS-NO03)	2**	1, 2, 3	33
d. Suppression Chamber Water Level - High (E41-LSH-NO15A,B)	2**	1, 2, 3	33
e. Bus Power Monitor# (E41-K55 and E41-K56)	1/bus	1, 2, 3	32
<u>4. AUTOMATIC DEPRESSURIZATION SYSTEM</u>			
a. Drywell Pressure - High, coincident with (E11-PT-NO10A,B,C,D) (E11-PTM-NO10A-1,B-1,C-1,D-1)	2	1, 2, 3	30
b. Reactor Vessel Water Level - Low, Level 3 (B21-LT-NO31A,B,C,D) (B21-LTS-NO31A-3,B-3,C-3,D-3)	2	1, 2, 3	30

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
<u>AUTOMATIC DEPRESSURIZATION SYSTEM (Continued)</u>			
c. Reactor Vessel Water Level - Low, Level 1 (B21-LT-NO42A,B) (B21-LTM-NO42A-1,B-1)	1	1, 2, 3	30
d. ADS Timer (B21-TDPU-K5A,B)	1	1, 2, 3	31
e. Core Spray Pump Discharge Pressure - High (Permissive) (E21-PS-NO08A,B and E21-PS-NO09A,B)	2	1, 2, 3	31
f. RHR (LPCI MODE) Pump Discharge Pressure - High (Permissive) (E11-PS-NO16A,B,C,D and E11-PS-NO20A,B,C,D)	2/pump	1, 2, 3	31
g. Bus Power Monitor# (B21-K1A,B)	1/bus	1, 2, 3	32

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
<u>5. LOSS OF POWER</u>					
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)	1/bus	1/bus	1/bus	1,2,3,4 <sup>##</sup> ,5 <sup>##</sup>	34
b. 4.16 kv Emergency Bus Undervoltage (Degraded Voltage)	3/bus	2/bus	2/bus	1,2,3,4 <sup>##</sup> ,5 <sup>##</sup>	35

\* Not applicable when two core spray system subsystems are OPERABLE per Specification 3.5.3.1.

\*\* Provides signal to HPCI pump suction valves only.

# Alarm only.

## Required when ESF equipment is required to be OPERABLE.

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<b>1. CORE SPRAY SYSTEM</b>		
a. Reactor Vessel Water Level - Low, Level 3 (B21-LTS-NO31A-4, B-4, C-4, D-4)	$\geq + 2.5$ inches*	$\geq + 2.5$ inches*
b. Reactor Steam Dome Pressure - Low (B21-PTS-NO21A-2, B-2, C-2, D-2)	$410 \pm 15$ psig	$410 \pm 15$ psig
c. Drywell Pressure - High (E11-PTS-NO11A-2, B-2, C-2, D-2)	$\leq 2$ psig	$\leq 2$ psig
d. Time Delay-Relay (E21-K16A, B)	$14 \leq t \leq 16$ secs	$14 \leq t \leq 16$ secs
e. Bus Power Monitor (E21-K1A, B)	NA	NA
<b>2. LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM</b>		
a. Drywell Pressure - High (E11-PTM-NO11A-1, B-1, C-1, D-1)	$\leq 2$ psig	$\leq 2$ psig
b. Reactor Vessel Water Level - Low, Level 3 (B21-LTS-NO31A-4, B-4, C-4, D-4)	$\geq + 2.5$ inches*	$\geq + 2.5$ inches*
c. Reactor Vessel Shroud Level (B21-LTM-NO36-1 and B21-LTM-NO37-1)	$\geq - 53$ inches*	$\geq - 53$ inches*
d. Reactor Steam Dome Pressure - Low		
1. RHR Pump Start and LCPI Valve Actuation (B21-PTS-NO21A-2, B-2, C-2, D-2)	$410 \pm 15$ psig	$410 \pm 15$ psig
2. Recirculation Pump Discharge Valve Actuation (B21-PTM-NO21A-1, B-1, C-1, D-1)	$310 \pm 15$ psig	$310 \pm 15$ psig

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM (Continued)</u>		
e. RHR Pump Start - Time Delay Relay (STR-2A1,2 and STR-2B1,2)	$9 < t < 11$ seconds	$9 < t < 11$ seconds
f. Bus Power Monitor (E11-K106A,B)	NA	NA
<u>3. HIGH PRESSURE COOLANT INJECTION SYSTEM</u>		
a. Reactor Vessel Water Level - Low, Level 2 (B21-LTS-NO31A-2,B-2,C-2,D-2)	$\geq + 112$ inches*	$\geq + 112$ inches*
b. Drywell Pressure - High (E11-PTS-NO11A-2,B-2,C-2,D-2)	$\leq 2$ psig	$\leq 2$ psig
c. Condensate Storage Tank Level - Low (E41-LS-N002; E41-LS-N003)	$\geq 23$ feet 4 inches	$\geq 23$ feet 4 inches
d. Suppression Chamber Water Level - High (E41-LSH-NO15A,B)	$\leq -2$ feet**	$\leq -2$ feet**
e. Bus Power Monitor (E41-K55 and E41-K56)	NA	NA
<u>4. AUTOMATIC DEPRESSURIZATION SYSTEM</u>		
a. Drywell Pressure-High (E11-PTM-NO10A-1,B-1,C-1,D-1)	$\leq 2$ psig	$\leq 2$ psig
b. Reactor Vessel Water Level - Low, Level 3 (B21-LTS-NO31A-3,B-3,C-3,D-3)	$\geq + 2.5$ inches*	$\geq + 2.5$ inches*
c. Reactor Vessel Water Level - Low, Level 1 (B21-LTM-NO42A-1,B-1)	$\geq + 162.5$ inches*	$\geq + 162.5$ inches*
d. ADS Timer (B21-TDPU-K5A,B)	$\leq 120$ seconds	$\leq 120$ seconds

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>AUTOMATIC DEPRESSURIZATION SYSTEM (Continued)</u>		
e. Core Spray Pump Discharge Pressure - High (E21-PS-N008A,B and E21-PS-N009A,B)	$\geq 100$ psig	$\geq 100$ psig
f. RHR (LPCI MODE) Pump Discharge Pressure - High (E11-PS-N016A,B,C,D and E11-PS-N020A,B,C,D)	$\geq 100$ psig	$\geq 100$ psig
g. Bus Power Monitor (B21-K1A,B)	NA	NA
<u>5. LOSS OF POWER</u>		
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)#	a. 4.16 kv Basis - $2940 \pm 161$ volts b. 120 v Basis - $84 \pm 4.6$ volts c. $< 10$ sec. time delay	$2940 \pm 315$ volts $84 \pm 9$ volts $< 10$ sec. time delay
b. 4.16 kv Emergency Bus Undervoltage (Degraded Voltage)	a. 4.16 kv Basis - $3727 \pm 9$ volts b. 120 v Basis - $106.5 \pm 0.25$ volts c. $10 \pm 0.5$ sec. time delay	$3727 \pm 21$ volts $106.5 \pm 0.60$ volts $10 \pm 1.0$ sec. time delay

# This is an inverse time delay voltage relay. The voltages shown are the maximum that will not result in a trip. Lower voltage conditions will result in decreased trip times.

\* Vessel water levels refer to REFERENCE LEVEL ZERO.

\*\* Suppression chamber water level zero is the torus centerline minus 1 inch.

TABLE 4.3.3-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<b>1. CORE SPRAY SYSTEM</b>				
a. Reactor Vessel Water Level - Low, Level 3 (B21-LT-NO31A,B,C,D) (B21-LTS-NO31A-4,B-4,C-4,D-4)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3, 4, 5 1, 2, 3, 4, 5
b. Reactor Steam Dome Pressure - Low (B21-PT-NO21A,B,C,D) (B21-PTS-NO21A-2,B-2,C-2,D-2)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3, 4, 5 1, 2, 3, 4, 5
c. Drywell Pressure - High (E11-PT-NO11A,B,C,D) (E11-PTS-NO11A-2,B-2,C-2,D-2)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3 1, 2, 3
d. Time Delay Relay (E21-K16A,B)	NA	R	R	1, 2, 3, 4, 5
e. Bus Power Monitor (E21-K1A,B)	NA	R	NA	1, 2, 3, 4, 5
<b>2. LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM</b>				
a. Drywell Pressure - High (E11-PT-NO11A,B,C,D) (E11-PTM-NO11A-1,B-1,C-1,D-1)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3 1, 2, 3
b. Reactor Vessel Water Level - Low, Level 3 (B21-LT-NO31A,B,C,D) (B21-LTS-NO31A-4,B-4,C-4,D-4)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3, 4*, 5* 1, 2, 3, 4*, 5*
c. Reactor Vessel Shroud Level (B21-LT-NO36 and B21-LT-NO37) (B21-LTM-NO36-1 and B21-LTM-NO37-1)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3, 4*, 5* 1, 2, 3, 4*, 5*

TABLE 4.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<u>LOW PRESSURE COOLANT INJECTION MODE OF RHR SYSTEM (Continued)</u>				
d. Reactor Steam Dome Pressure - Low (B21-PT-NO21A,B,C,D)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3, 4*, 5*
1. RHR Pump Start and LPCI Injection Valve Actuation (B21-PTS-NO21A-2,B-2,C-2,D-2)	D	M	M	1, 2, 3, 4*, 5*
2. Recirculation Loop Pump Discharge Valve Actuation (B21-PTM-NO21A-1,B-1,C-1,D-1)	D	M	M	1, 2, 3, 4*, 5*
e. RHR Pump Start - Time Delay Relay (STR-2A1,2 and STR-2B1,2)	NA	R	R	1, 2, 3, 4*, 5*
f. Bus Power Monitor (E11-K106A,B)	NA	R	NA	1, 2, 3, 4*, 5*
<u>3. HIGH PRESSURE COOLANT INJECTION SYSTEM</u>				
a. Reactor Vessel Water Level - Low, Level 2 (B21-LT-NO31A,B,C,D)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
(B21-LTS-NO31A-2,B-2,C-2,D-2)	D	M	M	1, 2, 3
b. Drywell Pressure - High (E11-PT-NO11A,B,C,D)	NA <sup>(a)</sup>	NA	R <sup>(b)</sup>	1, 2, 3
(E11-PTS-NO11A-2,B-2,C-2,D-2)	D	M	M	1, 2, 3
c. Condensate Storage Tank Level - Low (E41-LS-NO02; E41-LS-NO03)	NA	M	Q	1, 2, 3
d. Suppression Chamber Water Level - High (E41-LSH-NO15A,B)	NA	M	Q	1, 2, 3
e. Bus Power Monitor (E41-K55 and E41-K56)	NA	R	NA	1, 2, 3

TABLE 4.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<b>4. AUTOMATIC DEPRESSURIZATION SYSTEM</b>				
a. Drywell Pressure - High (E11-PT-N010A,B,C,D) (E11-PTM-N010A-1,B-1,C-1,D-1)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3 1, 2, 3
b. Reactor Vessel Water Level - Low, Level 3 (B21-LT-N031A,B,C,D) (B21-LTS-N031A-3,B-3,C-3,D-3)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3 1, 2, 3
c. Reactor Vessel Water Level - Low, Level 1 (B21-LT-N042A,B) (B21-LTM-N042A-1,B-1)	NA <sup>(a)</sup> D	NA M	R <sup>(b)</sup> M	1, 2, 3 1, 2, 3
d. ADS Timer (B21-TDPU-K5A,B)	NA	R	R	1, 2, 3
e. Core Spray Pump Discharge Pressure - High (E21-PS-N008A,B and E21-PS-N009A,B)	NA	M	Q	1, 2, 3
f. RHR (LPCI MODE) Pump Discharge Pressure - High (E11-PS-N016A,B,C,D and E11-PS-N020A,B,C,D)	NA	M	Q	1, 2, 3
g. Bus Power Monitor (B21-K1A,B)	NA	R	NA	1, 2, 3



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 97 TO FACILITY LICENSE NO. DPR-62

CAROLINA POWER & LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

DOCKET NO. 50-324

1.0 Introduction

By letter dated January 26, 1983, the Carolina Power & Light Company (the licensee) submitted proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant (BSEP), Unit 2. The proposed changes modify the Technical Specifications to apply to new analog (continuous measuring) instrumentation that was installed in Unit 1 as indicated in Amendment No. 60 dated December 12, 1983 and was installed in Unit 2 during the current shutdown. The analog instrumentation replaces certain pressure switches and will provide improved performance of trip functions for reactor protection system actuation, containment isolation, reactor core isolation cooling system isolation and emergency core cooling system actuation. In addition, miscellaneous typographical errors are corrected.

2.0 Evaluation

On November 19, 1979 the licensee submitted a request for Technical Specification changes to replace certain digital instrumentation with analog equipment. In response, the staff issued License Amendment Nos. 26 and 50 on March 14, 1980. The Safety Evaluation (SE) attached to those amendments found the proposed modifications to be technically acceptable and established a procedure whereby the licensee would periodically submit Technical Specifications changes to revise instrument designations with surveillance requirements for those systems that had been modified. The licensee now proposes certain instrument Technical Specification changes based on the previously issued SE. The technical acceptability of the instrument replacement has been previously reviewed and approved in that SE. That SE is hereby incorporated by reference. Furthermore, since the licensee is following the previously established periodic instrument change procedure, including change surveillance requirements, and the instrument designation changes do not involve substantive changes of the type usually associated with NRC approved changes, we find the proposed changes to be acceptable.

3.0 Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will

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not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### 4.0 Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: M. Grotenhuis

Dated: May 22, 1984 .