

November 12, 1982

Docket Nos. 50-325
and 50-324

Mr. E. E. Utley
Executive Vice President
Carolina Power & Light Company
P. O. Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment Nos. 51 and 76 to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of: (1) changes to the technical specifications in response to your submittal of September 29, 1982; and (2) changes to the licenses in response to your submittal of August 25, 1982.

These amendments: (1) change scram discharge volume vent and drain valve surveillance requirements to reflect an 18-month periodicity and to reflect installed instrumentation logic; and (2) change the licenses to reflect the new name of the facility co-owner formerly known as North Carolina Municipal Power Agency Number 3.

Copies of the related Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by
D. B. Vassallo

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 51 to DPR-71
2. Amendment No. 76 to DPR-62
3. Safety Evaluation
4. Notice

cc w/enclosures:
See next page

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F.R. NOTICE
&
AMENDMENT

OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:ORB	OELD		
SURNAME	S. Norris	J. VanVliet	D. Vassallo	G. Rainas	M. KARMA		
DATE	10/5/82	10/5/82 pr	10/5/82	10/8/82	10/9/82		

Mr. E. E. Utley
Carolina Power & Light Company

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 51
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Carolina Power & Light Company dated September 29, 1982 and August 25, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-71 is hereby amended by changing paragraphs 2.A and 2.C.(2) to read as follows:
 - 2.A This license applies to the Brunswick Steam Electric Plant, Unit 1, a boiling water reactor and associated equipment (the facility), owned by the Carolina Power & Light Company and North Carolina Eastern Municipal Power Agency and operated by Carolina Power & Light Company. The facility is located on the Cape

Fear River, near Southport in Brunswick County, North Carolina, and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 31) and the "Environmental Report" as supplemented and amended.

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 51 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 12, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 51

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Revise Appendix A by removing page 3/4 1-4 and replacing with identically numbered page.

REACTIVITY CONTROL SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION: (Continued)

2. If the inoperable control rod(s) is inserted:
 - a) Within one hour disarm the associated directional control valves either:
 - 1) Electrically, or
 - 2) Hydraulically by closing the drive water and exhaust water isolation valves.
 - b) Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.
- c. With more than 8 control rods inoperable, be in at least HOT SHUTDOWN within 12 hours.

SURVEILLANCE REQUIREMENTS

4.1.3.1.1 The scram discharge volume drain and vent valves shall be demonstrated OPERABLE at least once per 31 days by:*

- a. Verifying each valve to be open.
- b. Cycling each valve at least one complete cycle of full travel.

4.1.3.1.2 All withdrawn control rods not required to have their directional control valves disarmed electrically or hydraulically shall be demonstrated OPERABLE by moving each control rod at least one notch:

- a. At least once per 7 days when above the preset power level of the RWM and RSCS, and
- b. At least once per 24 hours when above the preset power level of the RWM and RSCS and any control rod is immovable as a result of excessive friction or mechanical interference.

4.1.3.1.3 All withdrawn control rods shall be determined OPERABLE by demonstrating the scram discharge volume drain and vent valves OPERABLE, when the reactor protection system logic is tested per Specification 4.3.1.2, by verifying that the drain and vent valves:

- a. Close within 30 seconds after receipt of a signal for control rods to scram, and
- b. Open when the scram signal is reset or the scram discharge volume trip is bypassed.

*These valves may be closed intermittently for testing under administrative control.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 76
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Carolina Power & Light Company dated September 29, 1982 and August 25, 1982 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-62 is hereby amended by changing paragraphs 2.A and 2.C.(2) to read as follows:
 - 2.A This license applies to Brunswick Steam Electric Plant Unit 2, a boiling water reactor and associated equipment (the facility), owned by the Carolina Power & Light Company and North Carolina Eastern Municipal Power Agency and operated by Carolina Power & Light Company. The facility is located on the Cape Fear River, near Southport in Brunswick County, North Carolina, and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 29)

and the "Environmental Report" as supplemented and amended (Supplements 1 through 7).

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 76, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 12, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 76

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Remove the following pages and replace with identically numbered pages.

2-4, 3/4 1-4

TABLE 2.2.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. Intermediate Range Monitor, Neutron Flux - High ⁽¹⁾ (C51-IRM-K601A,B,C,D,E,F,G,H)	< 120 divisions of full scale	< 120 divisions of full scale
2. Average Power Range Monitor (C51-APRM-CH.A,B,C,D,E,F)		
a. Neutron Flux - High, 15% ⁽²⁾	< 15% of RATED THERMAL POWER	< 15% of RATED THERMAL POWER
b. Flow-Biased Neutron Flux - High ⁽³⁾⁽⁴⁾	< (0.66 W + 54%)	< (0.66 W + 54%)
c. Fixed Neutron Flux - High ⁽⁴⁾	< 120% of RATED THERMAL POWER	< 120% of RATED THERMAL POWER
3. Reactor Vessel Steam Dome Pressure - High (B21-PS-N023A,B,C,D)	< 1045 psig	< 1045 psig
4. Reactor Vessel Water Level - Low, Level #1 (B21-LIS-N017A,B,C,D)	> 162.5 inches above instrument zero	> 162.5 inches above top fuel guide
5. Main Steam Line Isolation Valve - Closure ⁽⁵⁾ (B21-F022A,B,C,D; B21-F028A,B,C,D)	< 10% closed	< 10% closed
6. Main Steam Line Radiation - High (D12-RM-K603A,B,C,D)	< 3 x full power background	< 3.5 x full power background
7. Drywell Pressure - High (C72-PS-N002A,B,C,D)	< 2 psig	< 2 psig
8. Scram Discharge Volume Water Level - High (C12-LSH-N013A,B,C,D) (C12-LSH-4516A,B,C,D)	< 109 gallons	< 109 gallons

REACTIVITY CONTROL SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION: (Continued)

2. If the inoperable control rod(s) is inserted:
 - a) Within one hour disarm the associated directional control valves either:
 - 1) Electrically, or
 - 2) Hydraulically by closing the drive water and exhaust water isolation valves.
 - b) Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.
- c. With more than 8 control rods inoperable, be in at least HOT SHUTDOWN within 12 hours.

SURVEILLANCE REQUIREMENTS

4.1.3.1.1 The scram discharge volume drain and vent valves shall be demonstrated OPERABLE at least once per 31 days by:*

- a. Verifying each valve to be open.
- b. Cycling each valve at least one complete cycle of full travel.

4.1.3.1.2 All withdrawn control rods not required to have their directional control valves disarmed electrically or hydraulically shall be demonstrated OPERABLE by moving each control rod at least one notch:

- a. At least once per 7 days when above the preset power level of the RWM and RSCS, and
- b. At least once per 24 hours when above the preset power level of the RWM and RSCS and any control rod is immovable as a result of excessive friction or mechanical interference.

4.1.3.1.3 All withdrawn control rods shall be determined OPERABLE by demonstrating the scram discharge volume drain and vent valves OPERABLE, when the reactor protection system logic is tested per Specification 4.3.1.2, by verifying that the drain and vent valves:

- a. Close within 30 seconds after receipt of a signal for control rods to scram, and
- b. Open when the scram signal is reset or the scram discharge volume trip is bypassed.

*These valves may be closed intermittently for testing under administrative control.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 51 TO FACILITY LICENSE NO. DPR-71 AND
AMENDMENT NO. 76 TO FACILITY LICENSE NO. DPR-62
CAROLINA POWER & LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
DOCKET NOS. 50-325 AND 50-324

1.0 Introduction

By letter dated September 29, 1982, the Carolina Power & Light Company (licensee) proposed changes to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant Unit Nos. 1 and 2 (BSEP). The proposed changes revise the scram discharge volume (SDV) vent and drain valve surveillance requirements to reflect an 18-month periodicity in lieu of the 120-day periodicity required by present BSEP technical specifications.

By letter dated August 25, 1982, the licensee proposed changes to Facility Operating License Nos. DPR-71 and DPR-62 to reflect the new name of the co-owner formerly known as North Carolina Municipal Power Agency No. 3.

2.0 Scram Discharge Volume Vent and Drain Valves

2.1 Background

By letter dated July 7, 1980 we requested all operating BWR licensees to propose technical specification changes to provide, among other things, surveillance requirements for SDV vent and drain valves. Model technical specifications were enclosed with that letter to provide guidance to licensees. By letters dated November 13, 1980 and November 17, 1981 the licensee responded with proposed technical specifications. We reviewed, approved and subsequently issued these technical specifications in Amendment Nos. 48 and 72, dated August 5, 1982. Upon implementation, however, the licensee realized that the technical specifications contained an improper reference such that surveillance testing would be required essentially every 120 days in lieu of the 18-month periodicity cited in the model technical specifications and which we consider sufficient. Thus, the licensee now proposes to revise the technical specifications to permit an 18-month surveillance periodicity.

2.2 Evaluation

As evidenced by present Boiling Water Reactor Standard Technical Specifications and by the model technical specifications enclosed with our July 7, 1980 letter, we consider an 18-month periodicity to be sufficient to verify operability of the SDV vent and drain valves. Thus we consider the proposed changes to be acceptable.

3.0 North Carolina Municipal Power Agency Number 3 Name Change

3.1 Background and Evaluation

On December 16, 1981, the Board of Commissioners of the North Carolina Municipal Power Agency Number 3, co-owner of the Brunswick facility, acted to change its name to North Carolina Eastern Municipal Power Agency. The licensee now proposes to change the licenses to reflect this name change. Since the proposed change involves only a name change and not a change in ownership, we find it to be acceptable.

4.0 Environmental Considerations

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

5.0 Conclusions

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 12, 1982

Principal Contributor: J. A. Van Vliet

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-325 AND 50-324CAROLINA POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 51 and 76 to Facility Operating License Nos. DPR-71 and DPR-62 issued to Carolina Power & Light Company (the licensee) which revised the Technical Specifications and licenses for operation of the Brunswick Steam Electric Plant, Units 1 and 2 (the facility), located in Brunswick County, North Carolina. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications to lengthen scram discharge volume surveillance periodicities and revise the licenses to reflect the co-owner's recent name change.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

- 2 -

For further details with respect to this action, see (1) the applications for amendments dated September 29, 1982 and August 25, 1982 (2) Amendment Nos. 51 and 76 to License Nos. DPR-71 and DPR-62, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Southport-Brunswick County Library, 109 West Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 12th day of November, 1982

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing