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Docket Nos. 50-325

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Mr. J. A. Jones
Senior Executive Vice President
Carolina Power & Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment Nos. 45 and 68 to Facility Operating License Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant (BSEP), Unit Nos. 1 and 2, respectively. These amendments consist of changes to the Appendix B Technical Specifications in response to your application dated January 15, 1982.

The changes revise the Appendix B Technical Specifications to require release rates of noble gases, I-131 and radioactive materials in particulate form to be averaged and compiled on a calendar year basis in lieu of the previously required 12 consecutive months approach. The changes are in keeping with present Radiological Effluent Technical Specifications (RETS) implementation guidance for operating reactors. While the changes do provide some short-term relief for the BSEP Unit No. 2 gaseous effluent release problem, they do not provide long-term relief since the annual release rate limits remain unchanged. We feel that the long-term solution for this problem continues to be completion of the corrective actions described in your October 30, 1981 submittal, including replacement of all leaking fuel during the upcoming BSEP Unit No. 2 refueling outage, and installation of the Augmented Off-gas systems.

Copies of our Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED BY

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 45 to DPR-71
2. Amendment No. 68 to DPR-62
3. Safety Evaluation
4. Notice

cc: w/enclosures

See next page

FR NOTICE
& AMENDMENT

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OFFICE	ORB#2	ORB#2	ORB#2	ADLR	OELD	
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DATE	2/1/82	2/17/82	2/17/82	2/17/82	2/17/82	

Mr. J. A. Jones
Carolina Power & Light Company

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company dated January 15, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 45, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 19, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Revise the Appendix B Technical Specifications as follows:

<u>Remove</u>	<u>Insert</u>
2-11	2-11
2-12	2-12
2-20	2-20
2-21	2-21

2.0 ENVIRONMENTAL PROTECTION CONDITIONS

3.0 SURVEILLANCE REQUIREMENTS

2.5.2 Gaseous Waste Effluents (cont'd)

(2) The release rate limit of I-131 and radioactive materials in particulate form with half-lives greater than eight days, released to the environs as part of the gaseous wastes from the site shall be:

$$[3.7 \times 10^4]Q_s + [5.8 \times 10^6]Q_v \leq 1$$

where Q_s = release rate from the main stack in Ci/sec (as elevated release)

Q_v = release rate from the vents in Ci/sec (ground release)

- b. Should any of the conditions of Specifications 2.5.2.b(1) or 2.5.2.b(2) listed below be exceeded, the licensee shall make an investigation to identify the causes of the release rates, define and initiate a program of action to reduce the release rates to design objective levels listed in Sections 2.5.a and 2.5.b for gaseous wastes and report these actions to the Commission within 30 days from the end of the calendar year during which the releases occurred.

(1) The calculational method for determining the average release rate of noble gases from the site during any calendar year shall be:

$$\sum_{i=1}^n DAB_i [(4.0 \times 10^2)Q_{si} + (1.6 \times 10^5)Q_{vi}] \leq 1$$

and

$$\sum_{i=1}^n [DETA_i (1.9 \times 10^5)Q_{si} + DAG_i (3.2 \times 10^5)Q_{vi}] \leq 1$$

where Q_{si} = release rate of radioisotope i from the main stack in Ci/sec.

Q_{vi} = release rate of radioisotope i from the vents of each reactor in Ci/sec.

3.5.2 Gaseous Waste Effluents (cont'd)

- (2) within 1 month, following each refueling outage.
 (3) within 72 hours, if the gaseous waste monitors indicate an increase of greater than 50% in the steady state fission gas release after factoring out increases due to power changes.

- d. All waste gas effluent monitors shall be calibrated at least quarterly by means of a known radioactive source which has been calibrated to a National Bureau of Standards source. Each monitor shall have a functional test at least monthly and an instrument check at least daily excluding days of no discharge.
- e. Sampling and analysis of radioactive material in gaseous waste, particulate form, and radioiodine shall be performed in accordance with Table 3.5-2.

2.0 ENVIRONMENTAL PROTECTION CONDITIONS

3.0 SURVEILLANCE REQUIREMENTS

2.5.2 Gaseous Waste Effluents (cont'd)

DAB₁ = The beta air dose factor from Table 3.5-5 in mrad-m³/pCi-yr

DETA₁ = Gamma air elevated release dose factor from Table 3.5-5 in mrad/Ci

DAG₁ = The gamma air dose factor from Table 3.5-5 in mrad-m³/pCi-yr

(2) The calculational method for determining the average release rate from the site of I-131 and radioactive materials in particulate form with half-lives greater than eight days during any calendar year shall be:

$$(6.56 \times 10^6 Q_S) + (7.46 \times 10^7 Q_V) \leq 1$$

(If no teen, child or infant milk consumption)

The consumption of milk must be demonstrated by the Radiological Environmental Monitoring Program 4.2.7. If the Radiological Environmental Monitoring Program determines the consumption of milk by teen, child, or infant the above equation shall be modified by the appropriate coefficient (Dose Factors) of Regulatory Guide 1.109.

- c. Should any of the conditions of Specifications 2.5.2.c(1) or 2.5.2.c(2) listed below be exceeded, the licensee shall make an investigation to identify the causes of the release rates, define and initiate a program of action to reduce the release rates to design objective levels listed in Sections 2.5.a and 2.5.b for gaseous wastes and report these actions to the Commission within 30 days from the end of the calendar quarter during which the releases occurred.

3.5.2 Gaseous Effluents (cont'd)

- f. Calculations for meeting the requirements of Specifications 2.5.2.b(1) and (2) shall be performed at least once every 31 days.

infant via the cow-milk-infant pathway to less than or equal to 1500 mrem/year for the nearest cow to the plant.

For Specification 2.5.2.a(2), dose calculations have been made for the critical sectors and critical pathways for I-131 and radioactive material in particulate form with half-lives greater than eight days. The calculations consider site meteorology for these releases.

Specification 2.5.2.b establishes upper site levels for the releases of noble gases, iodines and particulates with half lives greater than eight days, and iodine-131 at the design objective annual quantity during any calendar year. Since BSEP does not have an AOG that has been demonstrated to be continuously operable, the content of these limiting conditions for operation assumes that the design objectives of 2.5a and b for gaseous wastes can be met. This specification does not limit the instantaneous gaseous radioactive release rate, but permits the licensee the flexibility of operation to assure that the public is provided a dependable source of power under unusual operating conditions which may temporarily result in higher releases than the objectives and yet remain below annual design objective releases. The equation limiting radioactivity releases was established based on on-site meteorological data and methodology of Regulatory Guides 1.109 and 1.111, and methods provided in Meteorology and Atomic Energy (1968).

For iodine-131 and radioactive material in particulate form with half-lives greater than eight days, the critical location for ground releases is the SSE sector distance of 1464 meters where X/Q is $6.5 \times 10^{-6} \text{ sec/m}^3$ for the dose due to inhalation. The critical location for elevated releases is the SSE sector at a distance of 1464 meters where the X/Q is $3.45 \times 10^{-8} \text{ sec/m}^3$ for the dose, due to inhalation. The assumptions for the grass-cow-milk-thyroid chain are listed in Table 3.5-6. The grass-cow-milk-thyroid chain is controlling.

The reporting requirements of 2.5.2.b and 2.5.2.c delineate that the cause be identified whenever the release of gaseous effluents exceeds the annual objective during any calendar year or one-half the annual objective quantity during any calendar quarter, and describe the proposed program of action to reduce such release rates to the design objectives.

Specifications 2.5.2.d and 2.5.2.e assure compliance with NRC general design criterion 64. The 24-hour period will allow an investigation of several hours to determine the cause of the monitor inoperability and possible repair prior to initiating the hot-shutdown.

Specification 2.5.2.f is to monitor the performance of the core. A sudden increase in the activity levels of gaseous releases may be the result of defective fuel. Since core performance is of utmost importance in the resulting doses, a report must be filed within 10 days following the specified increase in gaseous radioactive releases.

Specification 2.5.2.g requires that the primary containment atmosphere receive treatment for the removal of gaseous iodine and particulates prior to its release.

Specification 2.5.2.h requires that hydrogen concentration in the system shall be monitored at all times during AOG operation to prevent buildup of combustible concentrations.

The sampling and monitoring requirements given under Specification 3.5.2 provide assurance that radioactive materials released in gaseous wastes are properly controlled and monitored in conformance with the requirements of Design Criteria 60 and 64. These requirements provide the data for the licensee and the Commission to evaluate the plant's performance relative to radioactive wastes released to the environment. Reports on the quantities of radioactive materials released in gaseous effluents are furnished to the Commission on the basis of Section 5.4 of these Technical Specifications and in conformance with Regulatory Guide 1.21. On the basis of such reports and any additional information the Commission may obtain from the licensee or others, the Commission may from time to time require the licensee to take such action as the Commission deems appropriate.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company dated January 15, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 19, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 68

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Revise the Appendix B Technical Specifications as follows:

<u>Remove</u>	<u>Insert</u>
2-11	2-11
2-12	2-12
2-20	2-20
2-21	2-21

2.0 ENVIRONMENTAL PROTECTION CONDITIONS

3.0 SURVEILLANCE REQUIREMENTS

2.5.2 Gaseous Waste Effluents (cont'd)

(2) The release rate limit of I-131 and radioactive materials in particulate form with half-lives greater than eight days, released to the environs as part of the gaseous wastes from the site shall be:

$$[3.7 \times 10^4]Q_s + [5.8 \times 10^6]Q_v \leq 1$$

where Q_s = release rate from the main stack in Ci/sec (as elevated release)

Q_v = release rate from the vents in Ci/sec (ground release)

- b. Should any of the conditions of Specifications 2.5.2.b(1) or 2.5.2.b(2) listed below be exceeded, the licensee shall make an investigation to identify the causes of the release rates, define and initiate a program of action to reduce the release rates to design objective levels listed in Sections 2.5.a and 2.5.b for gaseous wastes and report these actions to the Commission within 30 days from the end of the calendar year during which the releases occurred.

(1) The calculational method for determining the average release rate of noble gases from the site during any calendar year shall be:

$$\sum_{i=1}^n DAB_i [(4.0 \times 10^2)Q_{si} + (1.6 \times 10^5)Q_{vi}] \leq 1$$

and

$$\sum_{i=1}^n [DETA_i (1.9 \times 10^5)Q_{si} + DAG_i (3.2 \times 10^5)Q_{vi}] \leq 1$$

where Q_{si} = release rate of radioisotope i from the main stack in Ci/sec.

Q_{vi} = release rate of radioisotope i from the vents of each reactor in Ci/sec.

3.5.2 Gaseous Waste Effluents (cont'd)

- (2) within 1 month, following each refueling outage.
 (3) within 72 hours, if the gaseous waste monitors indicate an increase of greater than 50% in the steady state fission gas release after factoring out increases due to power changes.

- d. All waste gas effluent monitors shall be calibrated at least quarterly by means of a known radioactive source which has been calibrated to a National Bureau of Standards source. Each monitor shall have a functional test at least monthly and an instrument check at least daily excluding days of no discharge.
- e. Sampling and analysis of radioactive material in gaseous waste, particulate form, and radioiodine shall be performed in accordance with Table 3.5-2.

2.0 ENVIRONMENTAL PROTECTION CONDITIONS

3.0 SURVEILLANCE REQUIREMENTS

2.5.2 Gaseous Waste Effluents (cont'd)

DAB₁ = The beta air dose factor from Table 3.5-5 in mrad-m³/pCi-yr

DETA₁ = Gamma air elevated release dose factor from Table 3.5-5 in mrad/Ci

DAG₁ = The gamma air dose factor from Table 3.5-5 in mrad-m³/pCi-yr

(2) The calculational method for determining the average release rate from the site of I-131 and radioactive materials in particulate form with half-lives greater than eight days during any calendar year shall be:

$$(6.56 \times 10^6 Q_S) + (7.46 \times 10^7 Q_V) \leq 1$$

(If no teen, child or infant milk consumption)

The consumption of milk must be demonstrated by the Radiological Environmental Monitoring Program 4.2.7. If the Radiological Environmental Monitoring Program determines the consumption of milk by teen, child, or infant the above equation shall be modified by the appropriate coefficient (Dose Factors) of Regulatory Guide 1.109.

- c. Should any of the conditions of Specifications 2.5.2.c(1) or 2.5.2.c(2) listed below be exceeded, the licensee shall make an investigation to identify the causes of the release rates, define and initiate a program of action to reduce the release rates to design objective levels listed in Sections 2.5.a and 2.5.b for gaseous wastes and report these actions to the Commission within 30 days from the end of the calendar quarter during which the releases occurred.

3.5.2 Gaseous Effluents (cont'd)

- f. Calculations for meeting the requirements of Specifications 2.5.2.b(1) and (2) shall be performed at least once every 31 days.

infant via the cow-milk-infant pathway to less than or equal to 1500 mrem/year for the nearest cow to the plant.

For Specification 2.5.2.a(2), dose calculations have been made for the critical sectors and critical pathways for I-131 and radioactive material in particulate form with half-lives greater than eight days. The calculations consider site meteorology for these releases.

Specification 2.5.2.b establishes upper site levels for the releases of noble gases, iodines and particulates with half lives greater than eight days, and iodine-131 at the design objective annual quantity during any calendar year. Since BSEP does not have an AOG that has been demonstrated to be continuously operable, the content of these limiting conditions for operation assumes that the design objectives of 2.5a and b for gaseous wastes can be met. This specification does not limit the instantaneous gaseous radioactive release rate, but permits the licensee the flexibility of operation to assure that the public is provided a dependable source of power under unusual operating conditions which may temporarily result in higher releases than the objectives and yet remain below annual design objective releases. The equation limiting radioactivity releases was established based on on-site meteorological data and methodology of Regulatory Guides 1.109 and 1.111, and methods provided in Meteorology and Atomic Energy (1968).

For iodine-131 and radioactive material in particulate form with half-lives greater than eight days, the critical location for ground releases is the SSE sector distance of 1464 meters where X/Q is 6.5×10^{-6} sec/m³ for the dose due to inhalation. The critical location for elevated releases is the SSE sector at a distance of 1464 meters where the X/Q is 3.45×10^{-8} sec/m³ for the dose, due to inhalation. The assumptions for the grass-cow-milk-thyroid chain are listed in Table 3.5-6. The grass-cow-milk-thyroid chain is controlling.

The reporting requirements of 2.5.2.b and 2.5.2.c delineate that the cause be identified whenever the release of gaseous effluents exceeds the annual objective during any calendar year or one-half the annual objective quantity during any calendar quarter, and describe the proposed program of action to reduce such release rates to the design objectives.

Specifications 2.5.2.d and 2.5.2.e assure compliance with NRC general design criterion 64. The 24-hour period will allow an investigation of several hours to determine the cause of the monitor inoperability and possible repair prior to initiating the hot-shutdown.

Specification 2.5.2.f is to monitor the performance of the core. A sudden increase in the activity levels of gaseous releases may be the result of defective fuel. Since core performance is of utmost importance in the resulting doses, a report must be filed within 10 days following the specified increase in gaseous radioactive releases.

Specification 2.5.2.g requires that the primary containment atmosphere receive treatment for the removal of gaseous iodine and particulates prior to its release.

Specification 2.5.2.h requires that hydrogen concentration in the system shall be monitored at all times during AOG operation to prevent buildup of combustible concentrations.

The sampling and monitoring requirements given under Specification 3.5.2 provide assurance that radioactive materials released in gaseous wastes are properly controlled and monitored in conformance with the requirements of Design Criteria 60 and 64. These requirements provide the data for the licensee and the Commission to evaluate the plant's performance relative to radioactive wastes released to the environment. Reports on the quantities of radioactive materials released in gaseous effluents are furnished to the Commission on the basis of Section 5.4 of these Technical Specifications and in conformance with Regulatory Guide 1.21. On the basis of such reports and any additional information the Commission may obtain from the licensee or others, the Commission may from time to time require the licensee to take such action as the Commission deems appropriate.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NO. DPR-71
AND AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NO. DPR-62
CAROLINA POWER AND LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 AND 50-324

Author: W. P. Gamill

INTRODUCTION

By letter dated January 15, 1982 Carolina Power and Light Company (licensee) requested a change to the Appendix B Environmental Technical Specifications (ETS) in licenses DPR-71 and 62 for the Brunswick Steam Electric Plant, Unit Nos. 1 and 2. Specification 2.5.2.b(1) defines the "calculational method for determining the average release rate of noble gases from the site during any 12 consecutive months." Specification 2.5.2.b(2) contains similar language and pertains to I-131 and radioactive materials in particulate form. In the event an annual limit is exceeded during any 12 consecutive months, the licensee must identify the causes of the release rates, define and initiate a program of action to reduce the release rates to design objective levels and report these actions to the Commission within 30 days from the end of the calendar quarter during which the releases occurred. The proposed change is to replace the "any 12 consecutive months" with "any calendar year."

DISCUSSION

The model Radiological Effluent Technical Specifications (RETS) for boiling water reactors (BWR), NUREG-0473, contain several action statements which require the licensee, in the event a quarterly effluent limit is exceeded, to take action "to reduce the releases...during the remainder of the calendar quarter and during the subsequent three calendar quarters so that the cumulative dose" does not exceed the annual limit. We met with a number of representatives of nuclear utilities under the auspices of the Atomic Industrial Forum, Inc., on November 10, 1981 to discuss the implementation of the RETS at operating reactors. Comments were made by several representatives to the effect that the above wording created an unnecessarily cumbersome recordkeeping and reporting requirement and suggested that a calendar year approach would be a more valid interpretation of the annual objectives in 10 CFR 50, Appendix B.

After considering these comments, we concluded that the present wording provided little or no increased protection of the public and that the suggested change should be accepted. Consequently, on November 20, 1981 we provided such guidance to our contractors who are responsible for resolving differences between the technical specifications for operating reactors and the model RETS. By memorandum from W. P. Gammill dated January 25, 1982, similar guidance was provided to the Effluent Treatment Systems Branch for its use in implementing the RETS for plants undergoing operating license review.

EVALUATION

Although the wording in the Brunswick technical specifications differs from that in the model RETS, the "12 consecutive month" requirement poses the same record keeping problem as did the model RETS. In light of the earlier decision on the model RETS, we consider the proposed technical specification change to be reasonable and consistent with present policy. Thus, we conclude that replacing the phrase "12 consecutive months" with the phrase "calendar year" in Brunswick Specification 2.5.2.b is acceptable.

It should be noted that Brunswick Unit No. 2 is continuing to encounter problems with higher than normal gaseous effluent releases. These problems were summarized in the Safety Evaluation which accompanied License Amendment Nos. 37 & 58 dated June 3, 1981. These amendments addressed the licensee's schedule for installing new augmented off-gas systems for Brunswick, Unit Nos. 1 and 2. Fission product leakage from the fuel has now increased to the point that the radioactive noble gas release rate is approaching twice the annual limit. The proposed change will provide the licensee some short-term relief since the transition will occur early in the calendar year. However, there will be no long-term relief since the annual release rate limits are unchanged. Refueling, now scheduled for May 1982, is expected to reduce releases to normal levels. However, depending upon fuel performance during the remainder of this cycle and following refueling, additional action may be required this calendar year.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §1.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: February 19, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-325 AND 50-324CAROLINA POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 45 and 68 to Facility Operating License Nos. DPR-71 and DPR-62 issued to Carolina Power & Light Company (the licensee) which revised Technical Specifications for operation of the Brunswick Steam Electric Plant, Unit Nos. 1 and 2 (the facility) located in Brunswick County, North Carolina. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications to require that, for radioactive effluent release rate recordkeeping, the licensee employ a calendar year approach in lieu of a 12 consecutive month approach.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of the amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendment dated January 15, 1982, (2) Amendment No. 45 to License No. DPR-71 and Amendment No. 68 to DPR-62, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the Southport-Brunswick County Library, 109 West Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 19th day of February 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing