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Docket No. 50-324

August 17, 1982

Mr. J. A. Jones
 Senior Executive Vice President
 Carolina Power & Light Company
 P. O. Box 1551
 Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment No. 73 to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant, Unit 2. The amendment consists of changes to the Technical Specifications in response to your submittal of June 16, 1982.

The amendment revises the Technical Specifications to reflect (1) the addition of diverse instrumentation to the scram discharge instrument volumes, and (2) the deletion of certain numbers resulting from removal of the control rod drive return line.

Copies of the Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Original signed by
 D. B. Vassallo

Domenic B. Vassallo, Chief
 Operating Reactors Branch #2
 Division of Licensing

Enclosures:

1. Amendment No. 73 to DPR-62
2. Safety Evaluation
3. Notice

cc w/enclosures:
 See next page

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*F.R. NOTICE
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 AMENDMENT*

OFFICE	DL:ORB#2 <i>sm</i>	DL:ORB#2 <i>fw</i>	DL:ORB#2	DL:OR	OELD		
SURNAME	S. Norris	J. VanVliet	D.B. Vassallo	G. Varnas	<i>N. KARMA</i>		
DATE	8/10/82	8/11/82 <i>pr</i>	8/11/82	8/11/82	8/17/82		

Mr. J. A. Jones
Carolina Power & Light Company

cc:

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Raleigh, North Carolina 27603

Southport - Brunswick County Library
109 W. Moore Street
Southport, North Carolina 28461

U. S. Environmental Protection Agency
Region IV Office
Regional Radiation Representative
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Atlanta, Georgia 30308

Resident Inspector
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James P. O'Reilly
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101 Marietta Street, Suite 3100
Atlanta, Georgia 30303



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company dated June 16, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 17, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Remove the following pages and replace with identically numbered pages.

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TABLE 3.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM (a)(b)</u>	<u>ACTION</u>
7. Drywell Pressure - High (C72-PS-N002 A,B,C,D)	1, 2 ^(e)	2	6
8. Scram Discharge Volume Water Level - High (C12-LSH-N013 A,B,C,D) (C12-LSH-4516A,B,C,D)	1, 2, 5 ^(f)	2	5
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	1 ^(g)	4	8
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSL-1756,1757,1758,1759)	1 ^(g)	2	8
11. Reactor Mode Switch in Shutdown Position (C72A-S1)	1, 2, 3, 4, 5	1	9
12. Manual Scram (C72A-S3A,B)	1, 2, 3, 4, 5	1	10

BRUNSWICK - UNIT 2

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Amendment No. 48, 73

TABLE 3.3.1-2

REACTOR PROTECTION RESPONSE TIMES

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>RESPONSE TIME</u> (Seconds)
1. Intermediate Range Monitors (C51-IRM-K601 A,B,C,D,E,F,G,H):	
a. Neutron Flux - High*	NA
b. Inoperative	NA
2. Average Power Range Monitor* (C51-APRM-CH.A,B,C,D,E,F):	
a. Neutron Flux - High, 15%	< 0.09
b. Flow-Biased Neutron Flux - High	NA
c. Neutron Flux - High, 120%	< 0.09
d. Inoperative	NA
e. Downscale	NA
f. LPRM	NA
3. Reactor Vessel Steam Dome Pressure - High (B21-PS-N023 A,B,C,D)	< 0.55
4. Reactor Vessel Water Level - Level #1 (B21-LIS-N017 A,B,C,D)	< 1.05
5. Main Steam Line Isolation Valve-Closure (B21-F022 A,B,C,D and B21-F028 A,B,C,D)	< 0.06
6. Main Steam Line Radiation - High (D12-RM-K603 A,B,C,D)	NA
7. Drywell Pressure - High (C72-PS-N002 A,B,C,D)	NA
8. Scram Discharge Volume Water Level - High (C12-LSH-N013 A,B,C,D) (C12-LSH-4516A,B,C,D)	NA
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	< 0.06
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSL-1756,1757,1758,1759)	< 0.08
11. Reactor Mode Switch in Shutdown Position (C72A-S1)	NA
12. Manual Scram (C72A-S3 A,B)	NA

*Neutron detectors are exempt from response time testing. Response time shall be measured from detector output or input of first electronic component in channel.

TABLE 4.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
8. Scram Discharge Volume Water Level - High (C12-LSH-N013 A,B,C,D) (C12-LSH-4516A,B,C,D)	NA	Q	R	1, 2, 5
9. Turbine Stop Valve - Closure (EHC-SVOS-1X,2X,3X,4X)	NA	M	R ^(h)	1
10. Turbine Control Valve Fast Closure, Control Oil Pressure - Low (EHC-PSL-1756,1757,1758,1759)	NA	M	R	1
11. Reactor Mode Switch in Shutdown Position (C72A-S1)	NA	R	NA	1, 2, 3, 4, 5
12. Manual Scram (C72A-S3A,B)	NA	Q	NA	1, 2, 3, 4, 5

- a. Neutron detectors may be excluded from CHANNEL CALIBRATION.
- b. Within 24 hours prior to start-up, if not performed within the previous 7 days.
- c. The IRM channels shall be compared to the APRM channels and the SRM instruments for overlap during each start-up, if not performed within the previous 7 days.
- d. When changing from CONDITION 1 to CONDITION 2, perform the required surveillance within 12 hours after entering CONDITION 2.
- e. This calibration shall consist of the adjustment of the APRM readout to conform to the power values calculated by a heat balance during CONDITION 1 when THERMAL POWER $>$ 25% of RATED THERMAL POWER.
- f. This calibration shall consist of the adjustment of the APRM flow-biased setpoint to conform to a calibrated flow signal.
- g. The LPRMs shall be calibrated at least once per effective full power month (EFPM) using the TIP system.
- h. This calibration shall consist of a physical inspection and actuation of these position switches.
- i. Instrument alignment using a standard current source.
- j. Calibration using a standard radiation source.

BRUNSWICK - UNIT 2

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Amendment No. 48, 73

TABLE 3.7.5-1 (Continued)

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE**</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
	<u>Reactor Water Cleanup System</u>			
2G31-1SS3	<u>Drywell</u> 54'	A	No	No
	<u>Condensate Drains System</u>			
2B21-51SS103	<u>Drywell</u> 29'	I	No	No
51SS105	26'	I	No	No
51SS106	18'	I	No	No
51SS109	31'	I	No	No
51SS111	28'	I	No	No
51SS113	23'	I	No	No
51SS115	24'	I	No	No
51SS118	24'	I	No	No
	<u>High Pressure Coolant Injection System</u>			
2E41-4SS44	<u>Drywell</u> 40'	I	No	No
4SS45	35'	I	No	No
4SS47	40'	I	No	No
4SS49	37'	I	No	No
4SS50	40'	I	No	No
4SS51	30'	I	No	No

BRUNSWICK - UNIT 2

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Amendment No. 80, 73



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. DPR-62
CAROLINA POWER & LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2
DOCKET NO. 50-324

1.0 Introduction

By letter dated June 16, 1982, the Carolina Power & Light Company (the licensee) submitted proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant (BSEP), Unit 2. The proposed changes revise the Technical Specifications to reflect (1) the addition of diverse instrumentation to the scram discharge instrument volumes (SDIVs), and (2) the deletion of certain snubbers resulting from removal of the control rod drive (CRD) return line.

2.0 Discussion and Evaluation

2.1 Scram Discharge Instrument Volume (SDIV) Diverse Instrumentation

During the 1982 BSEP Unit 2 refueling outage, the licensee installed additional, diverse instrumentation on the SDIVs. Installation of such instrumentation has been previously evaluated and approved as documented by the safety evaluation appended to an NRC generic letter dated December 9, 1980.

The licensee proposes to incorporate this additional instrumentation into the technical specifications. We have reviewed the proposed technical specification change and determined that the scram discharge volume (SDV) water level high operability, applicability, action and surveillance requirements remain unchanged, and that the proposed change only adds additional instrumentation. Therefore since the installation of SDIV diverse instrumentation has been previously approved and since the proposed change only adds additional SDV water level high instruments to the technical specifications, we find the proposed technical specification change to be acceptable.

2.2 Control Rod Drive (CRD) Return Line Snubbers

During the 1982 BSEP Unit 2 refueling outage, the licensee, in accordance with NUREG-0619 as forwarded by NRC generic letter dated November 13, 1980, cut and capped the CRD return line.

The licensee proposes to delete from the technical specifications those snubbers that supported the CRD return line prior to its removal. Since we are only concerned with snubbers that are required to insure the integrity of the reactor coolant system and safety related systems and since the snubbers in question no longer support a safety related system, we find the proposed technical specification change to be acceptable.

3.0 Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that:
(1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 17, 1982

Author: J. A. Van Vliet

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-324CAROLINA POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 73 to Facility Operating License No. DPR-62 issued to Carolina Power & Light Company (the licensee) which revised the Technical Specifications for operation of the Brunswick Steam Electric Plant, Unit 2 (the facility), located in Brunswick County, North Carolina. The amendment is effective as of the date of issuance.

The amendment revises the Technical Specifications to reflect (1) the addition of diverse instrumentation to the scram discharge instrument volumes, and (2) the deletion of certain shubbers resulting from removal of the control rod drive return line.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

- 2 -

For further details with respect to this action, see (1) the application for amendment dated June 16, 1982, (2) Amendment No. 73 to License No. DPR-62, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Southport Brunswick County Library, 109 West Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 17th day of August 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing