Docket

MAR 20 1981

Docket No. 50-324

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Mr. J. A. Jones
Senior Executive Vice President
Carolina Power & Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment No. 56 to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant (BSEP), Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application dated March 11, 1981.

The amendment establishes new vessel level setpoints that are consistent with the installation of a common reference level required by TMI Action Item II.K.3.27 in NUREG-0737.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

RECEIVED
MAR 25 1981
U.S. NUCLEAR REGULATORY COMMISSION

Enclosures:
1. Amendment No. 56 to DPR-62
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

F.R. NOTICE
+ AMENDMENT

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SURNAME	SNorris	JVanVliet	TNovak	KARMAN	TIppolito		
DATE	3/18/81	3/18/81	3/19/81	3/18/81	3/19/81		

NRC FORM 318 (10/80) NRCM 0240 **OFFICIAL RECORD COPY** ☆ USGPO: 1980—329-824

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 20, 1981

Docket No. 50-324

Mr. J. A. Jones
Senior Executive Vice President
Carolina Power & Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment No. 56 to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant (BSEP), Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application dated March 11, 1981.

The amendment establishes new vessel level setpoints that are consistent with the installation of a common reference level required by TMI Action Item II.K.3.27 in NUREG-0737.

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Sincerely,

Thomas A. Ippolito, Chief
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2. Safety Evaluation
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cc w/enclosures:
See next page

Mr. J. A. Jones
Carolina Power & Light Company

cc:

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Board of Commissioners
P. O. Box 249
Bolivia, North Carolina 28422

Denny McGuire (Ms)
State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, North Carolina 27603

Southport - Brunswick County Library
109 W. Moore Street
Southport, North Carolina 28461

Director, Criteria and Standards
Division
Office of Radiation Programs (ANR-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N. W.
Atlanta, Georgia 30308

Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 1057
Southport, North Carolina 28461

Mr. Charles R. Dietz
Plant Manager
P. O. Box 458
Southport, North Carolina 28461

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.56
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for Amendment by Carolina Power & Light Company dated March 11, 1981 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

   (2) The Technical Specifications contained in Appendices A and B, as revised through Amendment No.56 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 20, 1981

ATTACHMENT OT LICENSE AMENDMENT NO.56

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Remove the following pages and replace with identically numbered pages.

2-3/2-4
3/4 3-17/3/4 3-18
3/4 3-21/3/4 3-22
3/4 3-33/3/4 3-34
3/4 3-35/3/4 3-35a
3/4 3-63/3/4 3-64

The underlined page is an overleaf page and is provided for convenience.

2.2 LIMITING SAFETY SYSTEM SETTINGS

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

2.2.1 The reactor protection system instrumentation setpoints shall be set consistent with the Trip Setpoint values shown in Table 2.2.1-1.

APPLICABILITY: As shown for each channel in Table 3.3.1-1.

ACTION:

With a reactor protection system instrumentation setpoint less conservative than the value shown in the Allowable Values column of Table 2.2.1-1, declare the channel inoperable and apply the applicable ACTION statement requirement of Specification 3.3.1 until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value.

TABLE 2.2.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

FUNCTIONAL UNIT AND INSTRUMENT NUMBER	TRIP SETPOINT	ALLOWABLE VALUES
1. Intermediate Range Monitor, Neutron Flux - High(1) (C51-IRM-K601 A,B,C,D,E,F,G,H)	≤ 120 divisions of full scale	≤ 120 divisions of full scale
2. Average Power Range Monitor (C51-APRM-CH.A,B,C,D,E,F)		
a. Neutron Flux - High, 15%(2)	≤ 15% of RATED THERMAL POWER	≤ 15% of RATED THERMAL POWER
b. Flow Biased Neutron Flux - High(3)(4)	≤ (0.66 W + 54%)	≤ (0.66 W + 54%)
c. Fixed Neutron Flux - High(4)	≤ 120% of RATED THERMAL POWER	≤ 120% of RATED THERMAL POWER
3. Reactor Vessel Steam Dome Pressure - High (B21-PS-N023 A,B,C,D)	≤ 1045 psig	≤ 1045 psig
4. Reactor Vessel Water Level - Low, Level 1 (B21-LIS-N017 A,B,C,D)	≥ 162.5 inches above instrument zero	≥ 162.5 inches above instrument zero
5. Main Steam Line Isolation Valve - Closure(5) (B21-F022 A,B,C,D; B21-F028 A,B,C,D)	≤ 10% closed	≤ 10% closed
6. Main Steam Line Radiation - High (D12-RM-K603 A,B,C,D)	≤ 3 x full power background	≤ 3.5 x full power background
7. Drywell Pressure - High (C72-PS-N002 A,B,C,D)	≤ 2 psig	≤ 2 psig
8. Scram Discharge Volume Water Level - High (C12-LSH-N013 A,B,C,D)	≤ 109 gallons	≤ 109 gallons

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION AND INSTRUMENT NUMBER	TRIP SETPOINT	ALLOWABLE VALUE
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level - Low		
1. Level #1 (B21-LIS-N017 A,B,C,D)	≥ 162.5 inches	≥ 162.5 inches
2. Level #2 (B21-LIS-N024 A,B and B21-LIS-N025 A,B)	≥ 112 inches	≥ 112 inches
b. Drywell Pressure - High (C72-PS-N002 A,B,C,D)	≤ 2 psig	≤ 2 psig
c. Main Steam Line		
1. Radiation - High (D12-RM-K603 A,B,C,D)	≤ 3 x full power background	≤ 3.5 x full power background
2. Pressure - Low (B21-PS-N015 A,B,C,D)	≥ 825 psig	≥ 825 psig
3. Flow - High (B21-dPIS-N006 A,B,C,D; B21-dPIS-N007 A,B,C,D; B21-dPIS-N008 A,B,C,D; and B21-dPIS-N009 A,B,C,D)	≤ 140% of rated flow	≤ 140% of rated flow
4. Flow - High (B21-dPIS-N006A; B21-dPIS-N007B; B21-dPIS-N008C and B21-dPIS-N009D)	≤ 40% of rated flow	≤ 40% of rated flow
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010 A,B,C,D; B21-TS-N011 A,B,C,D; B21-TS N012 A,B,C,D; and B21-TS-N013 A,B,C,D)	≤ 200°F	≤ 200°F
e. Condenser Vacuum - Low (B21-PS-N056 A,B,C,D)	≥ 7 inches Hg vacuum	> 7 inches Hg vacuum
f. Turbine Building Area Temp - High (B21-TS-3225 A,B,C,D; B21-TS-3226 A,B,C,D; B21-TS-3227 A,B,C,D; B21-TS-3228 A,B,C,D; B21-TS-3229 A,B,C,D; B21-TS-3230 A,B,C,D; B21-TS-3231 A,B,C,D and B21-TS-3232 A,B,C,D)	≤ 200°F	≤ 200°F

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION AND INSTRUMENT NUMBER	TRIP SETPOINT	ALLOWABLE VALUE
2. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Building Exhaust Radiation - High (D12-RM-N010 A,B)	≤ 11 mr/hr	≤ 11 mr/hr
b. Drywell Pressure - High (C72-PS-N002 A,B,C,D)	≤ 2 psig	≤ 2 psig
c. Reactor Vessel Water Level - Low, Level #2 (B21-LIS-N024 A,B and B21-LIS-N025 A,B)	≥ 112 inches	≥ 112 inches
3. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. Δ Flow - High (G31-dFS-N603-1A,1B)	≤ 53 gal/min	≤ 53 gal/min
b. Area Temperature - High (G31-TS-N600A,B,C,D,E,F)	≤ 150°F	≤ 150°F
c. Area Ventilation Temperature ΔTemp-High (G31-TS-N602A,B,C,D,E,F)	≤ 50°F	≤ 50°F
d. SLCS Initiation (C41A-S1)	NA	NA
e. Reactor Vessel Water - Low, Level #2 (B21-LIS-N024A,B and B21-LIS-N025A,B)	≥ 112 inches	≥ 112 inches

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION AND INSTRUMENT NUMBER	TRIP SETPOINT	ALLOWABLE VALUE
5. SHUTDOWN COOLING SYSTEM ISOLATION		
a. Reactor Vessel Water - Low, Level #1 (B21-LIS-N017A,B,C,D)	≥ 162.5 inches	≥ 162.5 inches
b. Reactor Steam Dome Pressure - High (B32-PS-N018A,B)	≤ 140 psig	≤ 140 psig

TABLE 3.3.2-3

ISOLATION SYSTEM RESPONSE TIME

TRIP FUNCTION AND INSTRUMENT NUMBER	RESPONSE TIME (Seconds)
1. PRIMARY CONTAINMENT ISOLATION	
a. Reactor Vessel Water Level - Low	
1. Level #1 (B21-LIS-N017 A,B,C,D)	≤13
2. Level #2 (B21-LIS-N024 A,B and B21-LIS-N025 A,B)	≤1.0**
b. Drywell Pressure - High (C72-PS-N002 A,B,C,D)	≤13
c. Main Steam Line	
1. Radiation - High* (D12-RM-K603 A,B,C,D)	≤1.0**
2. Pressure - Low (B21-PS-N015 A,B,C,D)	≤13
3. Flow - High (B21-dPIS-N006 A,B,C,D; B21-dPIS-N007 A,B,C,D; B21-dPIS-N008 A,B,C,D and B21-dPIS-N009 A,B,C,D)	≤0.5**
4. Flow - High (B21-dPIS-N006A; B21-dPIS-N007B; B21-dPIS-B008C and B21-dPIS-N009D)	≤0.5**
d. Main Steam Line Tunnel Temperature - High (B21-TS-N010 A,B,C,D; B21-TS-N011 A,B,C,D; B21-TS-N012 A,B,C,D; and B21-TS-N013 A,B,C,D)	≤13
e. Condenser Vacuum - Low (B21-PS-N056 A,B,C,D)	≤13
f. Turbine Building Area Temperature - High (B21-TS-3225 A,B,C,D; B21-TS-3226 A,B,C,D; B21-TS-3227 A,B,C,D; B21-TS-3228 A,B,C,D; B21-TS-3229 A,B,C,D; B21-TS-3230 A,B,C,D; B21-TS-3231 A,B,C,D and B21-TS-3232 A,B,C,D)	NA
2. SECONDARY CONTAINMENT ISOLATION	
a. Reactor Building Exhaust Radiation - High* (D12-RM-N010 A,B)	≤13
b. Drywell Pressure - High (C72-PS-N002 A,B,C,D)	≤13
c. Reactor Vessel Water Level - Low, Level # 2 (B21-LIS-N024 A,B and B21-LIS-N025 A,B)	≤1.0**

*Radiation monitors are exempt from response time testing. Response time shall be measured from detector output or the input of the first electronic component in the channel.
**Isolation actuation instrumentation response time only.

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

ACTION

ACTION 30 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement:

a. For one trip system, place at least one inoperable channel in the tripped condition within one hour or declare the associated ECCS inoperable.

b. For both trip systems, declare the associated ECCS inoperable.

ACTION 31 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement, declare the associated ECCS inoperable.

ACTION 32 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement, verify bus power availability at least once per 12 hours or declare the associated ECCS inoperable.

ACTION 33 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement, place at least one inoperable channel in the tripped condition within one hour or declare the HPCS system inoperable.

ACTION 34 - With the number of OPERABLE channels less than the Total Number of Channels, declare the associated emergency diesel generator inoperable and take the ACTION required by Specification 3.8.1.1 or 3.8.1.2, as appropriate.

ACTION 35 - With the number of OPERABLE channels one less than the Total Number of Channels, place the inoperable channel in the tripped condition within 1 hour; operation may then continue until performance of the next required CHANNEL FUNCTIONAL TEST.

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION AND INSTRUMENT NUMBER	TRIP SETPOINT	ALLOWABLE VALUE
1. CORE SPRAY SYSTEM		
a. Reactor Vessel Water Level - Low, Level #3 (B21-LIS-N031A,B,C,D)	$\geq$ 2.5 inches	$\geq$ 2.5 inches
b. Reactor Steam Dome Pressure - Low (B21-PS-N021A,B,C,D)	410 $\pm$ 15 psig	410 $\pm$ 15 psig
c. Drywell Pressure - High (E11-PS-N011A,B,C,D)	$\leq$ 2 psig	$\leq$ 2 psig
d. Time Delay Relay	$14 \leq t \leq 16$ secs	$14 \leq t \leq 16$ secs
e. Bus Power Monitor (E21-K1A,B)	NA	NA
2. LPCI MODE OF RHR SYSTEM		
a. Drywell Pressure - High (E11-PS-N011A,B,C,D)	$\leq$ 2 psig	$\leq$ 2 psig
b. Reactor Vessel Water Level - Low, Level #3 (B21-LIS-N031A,B,C,D)	$\geq$ 2.5 inches	$\geq$ 2.5 inches
c. Reactor Vessel Shroud Level (B21-LITS-N036 and B21-LITS-N037)	$\geq$ 39" below TAF*	$\geq$ 39" below TAF*
d. Reactor Steam Dome Pressure - Low (B21-PS-N021A,B,C,D)		
1. RHR Pump Start and LCPI Valve Actuation	410 $\pm$ 15 psig	410 $\pm$ 15 psig
2. Recirculation Pump Discharge Valve Actuation	310 $\mp$ 15 psig	310 $\mp$ 15 psig
e. RHR Pump Start - Time Delay Relay	$9 \leq t \leq 11$ seconds	$9 \leq t \leq 11$ seconds
f. Bus Power Monitor (E11-K106A,B)	NA	NA

*Top of the active fuel.

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION AND INSTRUMENT NUMBER	TRIP SETPOINT	ALLOWABLE VALUE
3. HPCI SYSTEM		
a. Reactor Vessel Water Level - Low, Level #2 (B21-LIS-N031A,B,C,D)	≥ 112 inches	≥ 112 inches
b. Drywell Pressure-High (E11-PS-N011A,B,C,D)	≤ 2 psig	≤ 2 psig
c. Condensate Storage Tank Level - Low (E41-LS-N002, E41-LS-N003)	≥ 23'4"	≥ 23'4"
d. Suppression Chamber Water Level - High* (E41-LSH-N015A,B,)	≤ -2 feet	≤ -2 feet
e. Bus Power Monitor (E41-K55 and E41-K56)	NA	NA
4. ADS		
a. Drywell Pressure-High (E11-PS-N010A,B,C,D)	≤ 2 psig	≤ 2 psig
b. Reactor Vessel Water Level - Low, Level #3 (B21-LIS-N031A,B,C,D)	≥ 2.5 inches	≥ 2.5 inches
c. ADS Timer (B21-TDPU-K5A,B)	≤ 120 seconds	≤ 120 seconds
d. Core Spray Pump Discharge Pressure - High (E21-PS-N008A,B and E21-PS-N009A,B)	≥ 100 psig	≥ 100 psig
e. RHR (LPCI MODE) Pump Discharge Pressure - High (E11-PS-N016A,B,C,D and E11-PS-N020A,B,C,D)	≥ 100 psig	≥ 100 psig
f. Bus Power Monitor (B21-K1A,B)	NA	NA

*Suppression chamber water level zero is the torus centerline minus 1 inch.

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

FUNCTIONAL UNIT	TRIP VALUE	ALLOWABLE VALUES
5. LOSS OF POWER		
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)*	a. 4.16 kv Basis - 2940 ± 161 volts b. 120 v Basis - 84 ± 4.6 volts c. ≤ 10 sec. time delay	2940 ± 315 volts 84 ± 9 volts ≤ 10 sec. time delay
b. 4.16 kv Emergency Bus Undervoltage (Degraded Voltage)	a. 4.16 kv Basis - 3727 ± 9 volts b. 120 v Basis - 106.5 ± 0.25 volts c. 10 ± 0.5 sec. time delay	3727 ± 21 volts 106.5 ± 0.60 volts 10 ± 1.0 sec. time delay

*This is an inverse time delay voltage relay. The voltages shown are the maximum that will not result in a trip. Lower voltage conditions will result in decreased trip times.

TABLE 3.3.6.1-1

ATWS RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION

TRIP FUNCTION AND INSTRUMENT NUMBER	MINIMUM NUMBER OPERABLE TRIP SYSTEMS PER OPERATING PUMP
1. Reactor Vessel Water Level - Low Low, Level 2 (B21-LIS-N024 A, B; B21-LIS-N025, A, B)	1
2. Reactor Vessel Pressure-High (B21-PS-N045 A, B, C, D)	1

TABLE 3.3.6.1-2

ATWS RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION SETPOINTS

TRIP FUNCTION AND INSTRUMENT NUMBER	TRIP SETPOINT	ALLOWABLE VALUE
1. Reactor Vessel, Water Level - Low low, Level 2 (B21-LIS-N024 A, B; B21-LIS-N025 A, B)	≥ 112 inches	≥ 112 inches
2. Reactor Vessel Pressure-High (B21-PS-N045 A, B, C, D)	≤1120 psig	≤1120 psig

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-62

CAROLINA POWER AND LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-324

1.0 Introduction

By letter dated March 11, 1981 (Reference 1) Carolina Power and Light Company (the licensee) forwarded a proposed Technical Specification change that establishes revised vessel level setpoints that are consistent with a new common instrument zero level. The proposed common reference level is 367" above the vessel bottom. Establishment of the common zero level for all reactor vessel level instrumentation is called for as TMI Action Item II.K.3.27 in NUREG-0737 (Reference 2).

2.0 Evaluation

We have reviewed each of the proposed revised setpoints and find them to be consistent with the previously established safety settings. We also investigated the potential for operator error given that Unit 1 will not have the revised setpoints and operators are cross-assigned. To ensure that the proposed revised setpoints for Unit 2 do not create a potential for operator error, we require and CP&L has committed, by their letter dated March 18, 1981 (Reference 3), that all operators will be trained on the new level setpoints prior to completion of the modification on Unit 2. The required changes to operating and emergency procedures will be entered prior to operating with the new setpoints installed.

Since no change in actual water level for any function is involved in the proposed Technical Specification revisions, and since no instrumentation is being changed, we find the proposed Technical Specification revisions acceptable for use.

3.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 20, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-324

CAROLINA POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 56 to Facility Operating License No. DPR-62 issued to Carolina Power & Light Company (the licensee) which revised the Technical Specifications for operation of the Brunswick Steam Electric Plant, Unit No. 2 (the facility), located in Brunswick County, North Carolina. The amendment is effective as of the date of issuance.

The amendment establishes new vessel level setpoints that are consistent with the installation of a common reference level required by TMI Action Item II.K.3.27 in NUREG-0737.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated March 11, 1981, (2) Amendment No. 56 to License No. DPR-62, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Southport-Brunswick County Library, 109 West Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 20th day of March, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

T. A. Ippolito

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing