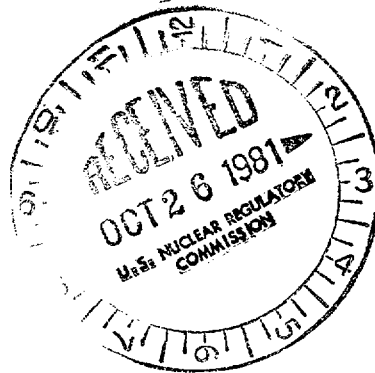


Dockets  
DMB-014

Docket Nos. 50-325  
50-324



OCT 20 1981

Mr. J. A. Jones  
Senior Executive Vice President  
Carolina Power & Light Company  
336 Fayetteville Street  
Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment Nos. 41 and 64 to Facility Operating Licenses Nos. DPR-71 and DPR-62 for the Brunswick Steam Electric Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal of September 24, 1981.

The amendments revise the Technical Specifications to reflect containment pressure instrumentation modifications performed in accordance with NUREG-0737, Action Item II.F.1.4. Since the Technical Specification changes merely revise instrument tag numbers to properly reflect plant nomenclature, they are considered to be purely administrative in nature. However, as specified by NUREG-0737, we will be performing a post-implementation review of your modifications at a later date.

Based on the foregoing, we have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

cp  
1

We have further concluded that the amendments do not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. They do not involve a significant increase in the probability or consequences of an accident, do not involve a significant decrease in a safety margin and, therefore, do not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

OFFICE							
SURNAME	8110280398	811020					
DATE	PDR	ADOCK	05000324				
	P		PDR				

Mr. J. A. Jones

2

A copy of the Notice of Issuance is also enclosed.

Sincerely,

ORIGINAL SIGNED BY

Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

- 1. Amendment No. 41 to DPR-71
- 2. Amendment No. 64 to DPR-62
- 3. Notice

cc: w/enclosures  
See next page

Distribution:

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*FR Notice  
+ Amendment  
only*

OFFICE	ORB#2	ORB#2	ORB#2	AD/OR	OELD		
SURNAME	SNorris	JVan Vliet	be Tippolito	TNovak	KARRIS		
DATE	10/9/81	10/9/81	10/13/81	10/13/81	10/14/81		

Mr. J. A. Jones  
Carolina Power & Light Company

cc:

Richard E. Jones, Esquire  
Carolina Power & Light Company  
336 Fayetteville Street  
Raleigh, North Carolina 27602

George F. Trowbridge, Esquire  
Shaw, Pittman, Potts & Trowbridge  
1800 M Street, N. W.  
Washington, D. C. 20036

Mr. Charles R. Dietz  
Plant Manager  
P. O. Box 458  
Southport, North Carolina 28461

Mr. Franky Thomas, Chairman  
Board of Commissioners  
P. O. Box 249  
Bolivia, North Carolina 28422

Mrs. Chrys Baggett  
State Clearinghouse  
Budget & Management  
116 West Jones Street  
Raleigh, North Carolina 27603

Southport - Brunswick County Library  
109 W. Moore Street  
Southport, North Carolina 28461

U. S. Environmental Protection Agency  
Region IV Office  
Regional Radiation Representative  
345 Courtland Street, N. W.  
Atlanta, Georgia 30308

Resident Inspector  
U. S. Nuclear Regulatory Commission  
P. O. Box 1057  
Southport, North Carolina 28461



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 41  
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power & Light Company dated September 24, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 41, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

8110280400 811020  
PDR ADDCK 05000324  
PDR

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 20, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 41

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Remove the following pages and replace with identically numbered pages.

3/4 3-51

3/4 3-52

TABLE 3.3.5.3-1

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>MINIMUM NO. OF OPERABLE INSTRUMENT CHANNELS</u>
1. Reactor Vessel Water Level (B21-LITS-NO26A,B; B21-LR-615; B21-LI-R604A,B; B21-LT-NO37 and B21-LTM-NO37-1)	2
2. Reactor Vessel Pressure (B21-PI-ROO4A,B; C32-LPR-R608 and C32-PT-NOO5A,B)	2
3. Containment pressure (CAC-PI-4176; CAC-PT-4176; CAC-PR-1257-1 and CAC-PT-4175)	2
4. Containment Temperature (CAC-TR-1258-1 thru 13,22,23,24 and C91-P602)	2
5. Suppression Chamber Atmosphere Temperature (CAC-TR-1258-17 thru 20 and C91-P602)	2
6. Suppression Chamber Water Level (CAC-LI-2601-3; CAC-LA-2602; CAC-LT-2601; CAC-LT-2602 and CAC-LY-2601-1)	2
7. Suppression Chamber Water Temperature (CAC-TR-1258-14, 21 and C91-P602)	2
8. Containment Radiation (CAC-AR-1260; CAC-AQH-1260-1,2,3; CAC-AR-1261; CAC-AQH-1261-1,2,3; CAC-AR-1262 and CAC-AQH-1262-1,2,3)	2
9. Containment Oxygen (CAC-AT-1259-2; CAC-AR-1259; CAC-AT-1263-2 and CAC-AR-1263)	2
10. Containment Hydrogen (CAC-AT-1959-1; CAC-AR-1259; CAC-AT-1263-1 and CAC-AR-1263)	2
11. Safety relief valve position indication: Primary - Sonic (B21-FY-4157 through 4167) Secondary - Temp. (B21-TR-R614 points 1-11)	1/valve

TABLE 4.3.5.3-1

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Water Level (B21-LITS-NO26A,B; B21-LR-R615; B21-LI-R604A,B; B21-LT-NO37 and B21-LTM-NO37-1)	M	R
2. Reactor Vessel Pressure (B21-PI-RO04A,B; C32-LPR-R608 and C32-PT-NO05A,B)	M	R
3. Containment pressure (CAC-PI-4176; CAC-PT-4176; CAC-PR-1257-1 and CAC-PT-4175)	M	R
4. Containment Temperature (CAC-TR-1258-1 thru 13,22,23,24 and C91-P602)	M	R
5. Suppression Chamber Atmosphere Temperature (CAC-TR-1258-17 thru 20 and C91-P602)	M	R
6. Suppression Chamber Water Level (CAC-LI-2601-3; CAC-LR-2602; CAC-LT-2601; CAC-LT-2602 and CAC-LY-2601-1)	M	R
7. Suppression Chamber Water Temperature (CAC-TR-1258-14, 21 and C91-P602)	M	R
8. Containment Radiation (CAC-AR-1260; CAC-AQH-1260-1,2,3; CAC-AR-1261; CAC-AQH-1261-1,2,3; CAC-AR-1262 and CAC-AQH-1262-1,2,3)	M	R
9. Containment Oxygen Concentration (CAC-AT-1259-2; CAC-AR-1259; CAC-AT-1263-2 and CAC-AR-1263)	M	R
10. Containment Hydrogen Concentration (CAC-AT-1259-1; CAC-AR-1259; CAC-AT-1263-1 and CAC-AR-1263)	M	R
11. Safety relief valve position indication (Primary-Sonic) (B21-FY-4157 thru 4167)	M	R
12. Safety relief valve position indication (Secondary-Temperature) (B21-TR-R614 points 1 thru 11)	M	R

BRUNSWICK - UNIT 1

3/4 3-52



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 64  
License No. DPR-62


1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power & Light Company dated September 24, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 64, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 20, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 64

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Remove the following pages and replace with identically numbered pages.

3/4 3-51

3/4 3-52

TABLE 3.3.5.3-1

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>MINIMUM NO. OF OPERABLE INSTRUMENT CHANNELS</u>
1. Reactor vessel water level (B21-LITS-NO26A,B; B21-LR-615; B21-LI-R604A,B and B21-LITS-NO37)	2
2. Reactor vessel pressure (B21-PI-RO04A,B; C32-LPR-R608 and C32-PT-NO05A,B)	2
3. Containment pressure (CAC-PI-4176; CAC-PT-4176; CAC-PR-1257-1 and CAC-PT-4175)	2
4. Containment Temperature (CAC-TR-1258-1 thru 13, 22, 23, 24 and C91-P602)	2
5. Suppression chamber atmosphere temperature (CAC-TR-1258-17 thru 20 and C91-P602)	2
6. Suppression chamber water level (CAC-LI-2601-3; CAC-LA-2602; CAC-LT-2601; CAC-LT-2602 and CAC-LY-2601-1)	2
7. Suppression chamber water temperature (CAC-TR-1258-14, 21 and C91-P602)	2
8. Containment radiation (CAC-AR-1260; CAC-AQH-1260-1, 2, 3; CAC-AR- 1261; CAC-AQH-1261-1, 2, 3; CAC-AR-1262 and CAC-AQH-1262-1, 2, 3)	2
9. Containment oxygen (CAC-AT-1259-2; CAC-AR-1259; CAC-AT-1263-2 and CAC-AR-1263)	2
10. Containment hydrogen (CAC-AT-1959-1; CAC-AR-1259; CAC-AT-1263-1 and CAC-AR-1263)	2
11. Safety relief valve position indication: Primary - Sonic (B21-FY-4157 thru 4167) Secondary - Temp. (B21-TR-R614 points 1-11)	1/valve

TABLE 4.3.5.3-1

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT AND INSTRUMENT NUMBER</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor vessel water level (B21-LITS-NO26A,B; B21-LR-R615; B21-LI-R604A,B and B21-LITS-NO37)	M	R
2. Reactor vessel pressure (B21-PI-RO04A,B; C32-LPR-R608 and C32-PT-NO05A,B)	M	R
3. Containment pressure (CAC-PI-4176; CAC-PT-4176; CAC-PR-1257-1 and CAC-PT-4175)	M	R
4. Containment temperature (CAC-TR-1258-1 thru 13, 22, 23, 24 and C91-P602)	M	R
5. Suppression chamber atmosphere temperature (CAC-TR-1258-17 thru 20 and C91-P602)	M	R
6. Suppression chamber water level (CAC-LI-2601-3; CAC-LR-2602; CAC-LT-2601; CAC-LT-2602 and CAC-LY-2601-1)	M	R
7. Suppression chamber water temperature (CAC-TR-1258-14, 21 and C91-P602)	M	R
8. Containment radiation (CAC-AR-1260; CAC-AQH-1260-1,2,3; CAC-AR-1261; CAC-AQH-1261-1,2,3; CAC-AR-1262 and CAC-AQH-1262-1,2,3)	M	R
9. Containment oxygen concentration (CAC-AT-1259-2; CAC-AR-1259; CAC-AT-1263-2 and CAC-AR-1263)	M	R
10. Containment hydrogen concentration (CAC-AT-1259-1; CAC-AR-1259; CAC-AT-1263-1 and CAC-AR-1263)	M	R
11. Safety relief valve position indication (Primary-Sonic) (B21-FY-4157 thru 4167)	M	R
12. Safety relief valve position indication (Secondary-Temperature) (B21-TR-R614 points 1 thru 11)	M	R

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-325 AND 50-324CAROLINA POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 41 and 64 to Facility Operating License Nos. DPR-71 and DPR-62 issued to Carolina Power & Light Company (the licensee) which revised the Technical Specifications for operation of the Brunswick Steam Electric Plant, Units Nos. 1 and 2 (the facility), located in Brunswick County, North Carolina. The amendments are effective as of the date of issuance.

The amendments consist of administrative changes to the Technical Specifications to properly reflect facility instrumentation nomenclature.

The application for amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of the amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendments dated September 24, 1981, (2) Amendment Nos. 41 and 64 to License Nos. DPR-71 and DPR-62, and (3) the Commission's letter to the licensee dated October 20, 1981. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Southport-Brunswick County Library, 109 West Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 20th day of October 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing