

Docket No. 50-324

October 27, 1977

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Carolina Power & Light Company
 ATTN: Mr. J. A. Jones
 Executive Vice President
 336 Fayetteville Street
 Raleigh, North Carolina 27602

Gentlemen:

The Commission has issued the enclosed Amendment No. 35 to Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your request dated October 11, 1977.

This amendment revises the Technical Specifications to permit the performance of the containment integrated leak rate test at full pressure, Pa.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by TWambach/for

A. Schwencer, Chief
 Operating Reactors Branch #1
 Division of Operating Reactors

Enclosures:

1. Amendment No. 35 to DPR-62
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosure:
 See next page

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OFFICE >	ORB#1	A. Schwencer	ASchwencer	W BUTLER	
x27433:tsb SURNAME >	CMTrammell				
DATE >	10/26/77	10/26/77	10/27/77	10/26/77	

October 27, 1977

cc: Richard E. Jones, Esquire
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Raleigh, North Carolina 27602

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Southport - Brunswick County Library
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Southport, North Carolina 28461

Mr. Steve J. Varnam
Chairman, Board of County
Commissioners of Brunswick County
Southport, North Carolina 28461

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Relations
116 West Jones Street
Raleigh, North Carolina 27603

Chief, Energy Systems
Analyses Branch (AW-459)
Office of Radiation Programs
U. S. Environmental Protection Agency
Room 645, East Tower
401 M Street, SW.
Washington, D.C. 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, NW.
Atlanta, Georgia 30308



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated October 11, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission,
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility License No. DPR-62 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 35 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for *J. V. Wambach*
A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 27, 1977

ATTACHMENT TO LICENSE AMENDMENT NO.35

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Revise Appendix A Technical Specifications by removing the following pages and replacing with identically numbered revised pages:

3.7-3/3.7-4

Marginal lines indicate changed area.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
<p data-bbox="196 369 683 436">3.7.A.2 <u>Containment Leak Rate Testing (Cont'd)</u></p> <p data-bbox="269 478 708 541">b. <u>Preoperational - Local Leak Rate Tests (LLRT)</u></p> <p data-bbox="347 573 756 888">The combined leakage rate of all penetrations and isolation valves shall not exceed 60 percent of the design basis accident leakage rate L_a. No one steam isolation valve shall have a leakage rate in excess of 11.5 scf/hr @ 25 psig.</p>	<p data-bbox="862 369 1349 436">4.7.A.2 <u>Containment Leak Rate Testing (Cont'd)</u></p> <p data-bbox="886 478 1341 541">b. <u>Preoperational - Local Leak Rate Tests (LLRT)</u></p> <ol style="list-style-type: none"> <li data-bbox="987 573 1406 699">(1) Primary containment testable penetrations shall be tested at 49 psig. <li data-bbox="987 730 1422 793">(2) Isolation valves shall be tested at 49 psig. <li data-bbox="987 825 1406 919">(3) Main steamline isolation valves shall be tested at 25 psig. <li data-bbox="987 951 1422 1014">(4) Bolted double-gasketed seals shall be tested. <li data-bbox="987 1045 1422 1140">(5) Personnel airlock seal shall be tested at a pressure of 49 psig. <li data-bbox="987 1171 1487 1686">(6) The drywell-suppression chamber vacuum breakers leakage shall be tested by pressurizing the drywell to approximately 1 psig and then measuring the subsequent suppression chamber pressure response. The test objective will be to detect any leak path between drywell and suppressing chamber whose capacity equals or exceeds that of a one-inch diameter orifice.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
<p data-bbox="203 302 683 363">3.7.A.2 <u>Containment Leak Rate Testing (Cont'd)</u></p> <p data-bbox="329 432 808 1913"> c.(1) The tests are acceptable if the leakage rate L_{tm} does not exceed 75 percent of the leakage rate L_t determined during the preoperational IPCLT as noted in Specification 3.7.A.2.a. (2) If any test fails to meet the criteria of (1) above, the schedule of subsequent tests shall be subject to review and approval of the AEC. (3) If two consecutive periodic tests fail to meet the acceptance criteria of (1) above, and IPCLT test shall be performed at each plant shutdown for refueling until two consecutive tests meet this acceptance criteria of (1) above, at which time the retest schedule in Specification 4.7.A.2.c will be resumed. (4) If repairs are necessary to meet the criteria of (1) above, the IPCLT test need not be repeated provided local measured leakage reductions achieved by repairs of individual leaks reduce the overall measured leakage rate sufficiently to meet the acceptance criteria. </p>	<p data-bbox="841 302 1321 363">4.7.A.2 <u>Containment Leak Rate Testing (Cont'd)</u></p> <p data-bbox="927 432 1463 1031"> c. After the initial preoperational leaking rate test, two integrated leak rate tests shall be performed at approximately equal intervals between the major shutdowns for inservice inspection conducted at 10-year intervals. In addition, an integrated leakage rate test shall be performed at the end of the 10-year interval, which may coincide with the inservice inspection shutdown period. All postoperational testing shall be conducted in accordance with the test program of Appendix J of 10CFR50. </p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 35 TO LICENSE NO. DPR-62

CAROLINA POWER AND LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-324

Introduction

By letter dated October 11, 1977, Carolina Power & Light Company (the licensee) requested a change to the Technical Specifications appended to the Brunswick Steam Electric Plant, Unit No. 2 Operating License. This change would permit the performance of the containment leak rate test at full pressure Pa, one of the options recognized in Appendix J of 10 CFR 50.

Discussion and Evaluation

The existing Brunswick 2 Technical Specifications state that post operational leak testing of the containment shall be performed at reduced pressure, one of the options allowed by Appendix J of 10 CFR 50.

The licensee has proposed that the words "reduced pressure" be deleted so that the containment integrated leak rate test can be performed at either a reduced pressure or at the peak pressure related to the design basis accident.

Testing at the peak pressure does not constitute a hazard to the facility since the drywell and suppression chamber are designed for 62 psig, while the peak test pressure, Pa, is calculated to be 49 psig. Furthermore, the containment preoperational tests were conducted at the peak test pressure.

Since full pressure tests at Pa are allowed by Appendix J, and have been successfully performed at Pa, and since Pa is less than design pressure we conclude that this change is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 27, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-324

CAROLINA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 35 to Facility Operating License No. DPR-62, issued to Carolina Power and Light Company (the licensee), which revised Technical Specifications for operation of Brunswick Steam Electric Plant, Unit No. 2 (the facility) located in Brunswick County, North Carolina. The amendment is effective as of the date of issuance.

This amendment revises the Technical Specifications to permit the performance of the containment integrated leak rate test at full pressure, Pa.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

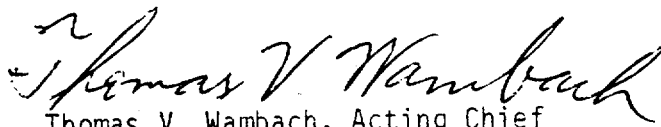
The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or

negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated October 11, 1977, (2) Amendment No. 35 to License No. DPR-62, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Southport-Brunswick County Library, 109 West Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 27th day of October 1977.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas V. Wambach, Acting Chief
Operating Reactors Branch #1
Division of Operating Reactors