MAR 21 1977

Docket No. 50-324

Carolina Power & Light Company ATTN: Mr. J. A. Jones.

Executive Vice President

336 Fayetteville Street

Raleigh, North Carolina 27602

Gentlemen:

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The Commission has issued the enclosed Amendment No. 25 to Facility Operating License No. DPR-62 for the Brunswick Steam Electric Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your request dated February 1, 1977.

This amendment modifies the inspection schedule for inaccessible safety-related shock suppressors (snubbers) and corrects administrative errors on the list of shock suppressors for which operability and inspection requirements exist.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Original signed by

A. Schwencer. Chief Operating Reactors Branch #1 Division of Operating Reactors

Enclosures:

1. Amendment No. 25 to DBR-62

2. Safety Evaluation

Federal Register Notice

cc w/encl: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

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Carolina Power & Light Company ATTN: Mr. J. A. Jones, Executive Vice President 336 Fayetteville Street Raleigh, North Carolina 27602

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Sincerely,

A.Schwencer, Chief

1. Sturentez-

Operating Reactors Branch #1
Division of Operating Reactors

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1. Amendment No. 25to DPR-62

Safety Evaluation

3. Federal Register Notice

cc w/encl:
See next page

cc: Richard E. Jones, Esquire Carolina Power & Light Company 336 Fayetteville Street Raleigh, North Carolina 27602

> George F. Trowbridge, Esquire Shaw, Pittman, Potts & Trowbridge 1800 M Street, NW Washington, D. C. 20036

> John J. Burney, Jr., Esquire Burney, Burney, Sperry & Barefoot 110 North Fifth Avenue Wilmington, North Carolina 28401

Southport - Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

Mr. Steve J. Varnam Chairman, Board of County Commissioners of Brunswick County Southport, North Carolina 28461

Office of Intergovernmental Relations 116 West Jones Street Raleigh, North Carolina 27603

Chief, Energy Systems
Analyses Branch (AW-459)
Office of Radiation Programs
U.S. Environmental Protection Agency
Room 645, East Tower
401 M Street, SW.
Washington, D.C. 20460

U.S. Environmental Protection Agency Region IV Office ATTN: EIS COORDINATOR 345 Courtland Street, NW. Atlanta, Georgia 30308



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25 License No. DPR-62

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated February 1, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-62 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 25, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief Operating Reactors Branch #1 Division of Operating Reactors

Attachment: Changes to the Technical Specifications

Date of Issuance: March 21, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 25 FACILITY OPERATING LICENSE NO. DPR-62 DOCKET NO. 50-324

Revise Appendix A as follows:

Remove the following pages and replace with identically numbered revised pages:

3.5-11c

3.5-20

3.5-27

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.5.J Shock Suppressors (Snubbers) on Safety Related Systems

cluded with the next License Amendment request.

5. Modifications to Table 3.5-1 due to changes in high radiation areas (plant shutdown) may be made and shall be submitted to NRC as part of the next License Amendment request.

4.5.J Shock Suppressors (Snubbers) on Safety Related Systems

propylene or other material that has been demonstrated to be compatible with the operating environment shall be visually inspected for operability every 31 days.

4. The initial inspection for accessible snubbers shall be performed within 6 months from November 15, 1976.

For the purpose of entering the schedule in Specification 4.5.J.1 for accessible snubbers it shall be assumed that the facility had been on a 6 month inspection interval.

The inaccessible snubbers shall begin on a 12 month inspection interval beginning on November 1, 1976.

Once each refueling cycle, a 5. representative sample of 10 hydraulic snubbers or approximately 10% of the hydraulic snubbers, whichever is less, shall be functionally tested for operability including verification of proper piston movement, lock up and bleed. For each unit and subsequent unit found inoperable, an additional 10% or ten hydraulic snubbers whichever is less shall be so tested until no more failures are found or all units have been tested. Snubbers of rated capacity greater than 50,000 lb. need not be functionally tested.

Table 3.5-1 (Con'd)

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION		ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
	Reactor Water	Cleanup Syst	em		
				No	No
2G31-1SS3	Drywe11	541	A A	No	No
50SS8		40'	A	No	No
50889		40'		No	No
698879		86'	A	No	No
678884		88'	A	No	No
138895		85 '	A	No	No
2SS98		86'	A	No	No
388100		84 '	A	No	No (
8SS155		60 '	A	No	No
8SS156		89 '	A	No	No
44SS168		77 '	A		No
65SS170		81'	Α	No	No
4SS176		77 '	• А	No	No
		65'	Α	No	No
5188186		67'	, A	No	No
5188187		2'	Α	No	No
51SS194 51SS196		91	A ·	No)\O
	Condensate Di	rains System			
			<u>_</u>	No	No
2B21-51SS103	Drywell	29'	I	No	No
51SS105	- · 3	26'	I	No No	Мо
51SS105 51SS106		18'	I		No
5188109		31'	I	No	No
		28 '	· I	No	· No
5188111		23'	I	No	No
5188113		24'	I	No	No
51SS115 51SS118		24'	Ι	No	NO
	Control Rod	Drive System	-		
				No	No
2C12-16SS1	Drywell	69'	Ī	No	No
16886	. •	68 '	I ~	. No	No
16SS7		691	I	No	No
16558		70'	ĭ	140	- · ·

Table 3.5-1 (Con'd)

SNUBBER	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION) วง	ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
<u>NO,</u>			Λ	No	No
588152	82		A	No	No
59SS159	84		A	NO	• •
	Reactor Recirculation Sys				No
2B32-SSA1		3'	Ι	No	No No
SSB1		3'	I	No	
SSA2	11		Ι	No	No
SSB2	11		I	No	No No
SSA3	11	l '	I	No	No
SSB3	11	1'	· I	No	No
SSA4	21	1'	I	Ио	No
SSB4	21	1'	I	No	No
SSA5	21	1'	I	No	No
SSB5	21	1'	I	No	No
SSA6	27	7 '	I	No	No
SSB6	. 27		Ι	No	No
SSB9A		0'	I	No	No
		01	I	No	No
SSB9B		41	I	No	No
SSA10		1'	I	No	No -
SSA11		1'	I	No	No
SSB11		0'	I	No	No
SSA12A		o'	Ï	No	No
SSA12B		0'	Ī	No	No
SSB12A		0'	Ī	No ·	No ·
SSB12B	•	21	Ī	No	No
2-PS-3705		32'	Ī	No	No
3706		32'	Ĩ	No	No
3707		32 '	Ī	No	No
3708		32'	I	No	No
3709		32'	Ï	No	No
3710			1	•••	
	Reactor Feedwater System	1 10 !	т	No	No
2B21-2SS3		38'	I	No	No
2884		56'		No	No
3SS6		11'	Ī	No	No
3SS9	3	39'	I	No	No
3SS11		41'	I	No	No
3S\$12		40'	I		No.
38813		51'	I	No	No
58817		38 '	I	No	No
5SS18		56'	I	No No	No
6SS20	4	41'	I	No	NO



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 25 TO LICENSE NO. DPR-62

CAROLINA POWER & LIGHT COMPANY

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-324

Introduction

By letter dated February 1, 1977, Carolina Power & Light Company (the licensee) requested changes to the Technical Specifications appended to Facility Operating License No. DPR-62 for operation of Brunswick Steam Electric Plant, Unit No. 2. The proposed changes relate to requirements for inspection of safety-related shock suppressors (snubbers). Specifically, the change would correct administrative errors in the present list of snubbers identified in Table 3.5-1 of the Technical Specifications, and would change the initial inspection interval for snubbers that are inaccessible during normal operation from 6 months to 12 months.

Evaluation

The proposed changes to Table 3.5-1 (snubber list) would (1) add several recirculation system snubbers inadvertently omitted when the Table was originally issued on November 15, 1976, and (2) change the classification of the snubbers on the reactor water cleanup system from "inaccessible" to "accessible" to correctly reflect access for inspection during reactor operation. These changes would correct errors currently present in Table 3.5-1 and are acceptable.

The balance of the licensee's proposal concerns the inspection schedule for inaccessible snubbers. Present Technical Specifications relating to inaccessible snubber inspection contain an inspection schedule that requires more frequent inspections as a function of the number of observed inoperable snubbers, and less frequent inspections if the number of observed failures has been low. Specifically, if one inoperable snubber is observed during the initial six month inspection interval, the next required inspection interval would be 12 months $(\pm 25\%)$.

During a scheduled maintenance outage in late October 1976, the licensee performed a complete inspection of all inaccessible snubbers. One inoperable snubber was found. The inspection was performed in accordance with the criteria of the present Technical Specifications, which, however, were not issued to the licensee until November 15, 1976 (Amendment No. 22), approximately two weeks after the inspection. The inspection was performed approximately 6 months following the previous inspection (which was performed in May 1976). The May 1976 inspection was a partial inspection, in that not all inaccessible snubbers were inspected at that time. This would normally have made the November inspection results less favorable, since some snubbers were in service more than 6 months before examination.

Based on this experience, it is concluded that the requirements of the initial snubber inspection program with regard to the inaccessible snubbers have been met-namely, 6 months of inaccessible snubber service with only one failure. We therefore conclude that the licensee's proposal to advance the inspection interval for inaccessible snubbers to 12 months is acceptable. In fact, had these Technical Specifications been in force earlier, the licensee would have been able to change the inspection schedule without this license amendment request. The essence of this change, therefore, is to adjust the inspection schedule for inaccessible snubbers to reflect the results of the inspection program that was actually in progress before the snubber inspection requirements were placed in effect by Amendment No. 22. The basic surveillance and operability requirements for snubbers are unaltered by this action.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, or negative declaration and environmental appraisal need not be prepared in connection with the issuance of this amendment.

<u>Conclusion</u>

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 21, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-324

CAROLINA POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 25 to Facility Operating License No. DPR-62, issued to Carolina Power & Light Company (the licensee), which revised Technical Specifications for operation of the Brunswick Steam Electric Plant, Unit No. 2 (the facility) located in Brunswick County, North Carolina. The amendment is effective as of its date of issuance.

This amendment modifies the inspection schedule for inaccessible safety-related shock suppressors (snubbers) and corrects administrative errors on the list of shock suppressors for which operability and inspection requirements exist.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR \$51.5(d)(4) an environmental impact statement or negative

declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated February 1, 1977, (2) Amendment No. 25 to License No. DPR-62, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C. and at the Southport Burnswick County Library, 109 W. Moore Street, Southport, North Carolina 28461. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 21st day of March 1977.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief
Operating Reactors Bra

Operating Reactors Branch #1
Division of Operating Reactors