

Dockets Nos. 50-825
and 50-324

JUN 20 1977

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Carolina Power & Light Company
ATTN: Mr. J. A. Jones
Executive Vice President
336 Fayetteville Street
Raleigh, North Carolina 27602

Gentlemen:

The Commission has issued the enclosed Amendment No. 6 to Facility Operating License No. DPR-71 for Brunswick Steam Electric Plant Unit No. 1, and Amendment No. 28 to Facility Operating License No. DPR-62 for Brunswick Steam Electric Plant Unit No. 2. The amendments consist of changes to the Technical Specifications in response to your request dated March 21, 1977.

The amendments revise the off-site corporate organization for facility management and technical support, and make several editorial changes to the administrative sections of the Technical Specifications to reflect the revised organization.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

ISI

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No. 6 to DPR-71
2. Amendment No. 28 to DPR-62
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:

See next page

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OFFICE ▶	ORB#1	OELD	ORB#1		
SURNAME ▶	CMTrammell		ASchwencer		
DATE ▶	4/19/77	4/ /77	4/6 /77		

June 20, 1977

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Mr. Steve J. Varnam
Chairman, Board of County
Commissioners of Brunswick County
Southport, North Carolina 28461

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Region IV Office
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated March 21, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility License No. DPR-71 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A, A-Prime, and B, attached hereto, as revised through Amendment No. 6, are hereby incorporated in the license. Appendix A shall be effective from the date of issuance of Unit 1 operating license until the Appendix A-Prime becomes effective on or before the initial criticality of Brunswick Unit 2 following its initial refueling outage. Carlina Power & Light Company shall operate the facility in accordance with the Technical Specifications as indicated above. The licensee shall inform the Office of Inspection and Enforcement, Region II, of the date that the Appendix A-Prime becomes effective.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 20, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 6

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Revise Appendix A, A-Prime, and B Technical Specifications by removing the following pages and replacing with identically numbered revised pages:

Appendix A

6.5-4
6.5-6
6.5-7
6.5-8
6.7-1
Fig. 6.2-1

Appendix A-Prime

6-2
6-8
6-11
6-12

Appendix B

5-1
Fig. 5.1-1

6.5.2 Independent Off-Site Safety Review Program
 Activities occurring during plant operations shall be independently reviewed as specified in succeeding paragraphs.

6.5.2.1 Purpose

The purpose of the independent off-site safety review program is to review significant plant changes, tests, and procedures; verify that reportable occurrences are promptly investigated and corrected in a manner which reduces the probability of recurrence of such events; and detect trends which may not be apparent to a day-to-day observer.

6.5.2.2 Responsibility

The Manager - Corporate Nuclear Safety under the Department Head - System Planning and Coordination is charged with the overall responsibility for administering the independent off-site nuclear safety review program as follows:

- a. Approves selection of the person or persons to conduct off-site safety reviews.
- b. Has access to the plant operating records and operating personnel in performing the independent reviews.
- c. Prepares and retains written records of reviews.
- d. Assures independent safety review is conducted on all items required by Section 6.5.2.4.
- e. Distributes reports and other records to appropriate managers.

6.5.2.3 Personnel

- a. Personnel assigned responsibility for independent reviews shall be specified in technical disciplines, and shall collectively have the experience and competence required to review problems in the following areas:
 1. Nuclear power plant operations
 2. Nuclear engineering
 3. Chemistry and radiochemistry
 4. Metallurgy
 5. Instrumentation and control
 6. Radiological safety

BSEP-1 & 2

technical specifications or an unreviewed safety questions pursuant to 10 CFR 50.59(c). Matters of this kind shall be referred to the Corporate Nuclear Safety Section by the Plant Nuclear Safety Committee following its review, or by other functional organizational units within Carolina Power & Light Company (CP&L) prior to implementation.

- c. Changes in the technical specifications or license amendments relating to nuclear safety prior to implementation, except in those cases where the change is identical to a previously reviewed proposed change.
- d. Violations, deviations and reportable events, which require reporting to the NRC within 24 hours, and as defined in the plant technical specifications such as:
 - (1) Violations of applicable codes, regulations, order, technical specifications, license requirements or internal procedures or instructions having safety significance; and
 - (2) Significant operating abnormalities or deviations from normal or expected performance of plant safety-related structures, systems, or components.

Review of events covered under this paragraph shall include the results of any investigations made and the recommendations resulting from such investigations to prevent or reduce the probability of recurrence of the event.

- e. Any other matter involving safe operation of the nuclear power plant which the Manager - Corporate Nuclear Safety Section deem appropriate for consideration, or which is referred to the Manager - Corporate Nuclear Safety Section by the onsite operating organization or by other functional organizational units within Carolina Power & Light Company.

6.5.2.5 Follow-Up Action

Results of Corporate Nuclear Safety (CNS) Section reviews, including recommendations and concerns will be documented.

- a. Copies of the documented review will be retained in the Corporate Nuclear Safety Section files.
- b. Recommendations and concerns will be submitted to the Manager - Nuclear Generation within 14 days of determination.
- c. A summation of Corporate Nuclear Safety Section recommendations and concerns will be submitted to the Company Chief Executive Officer;

Company Chief Operating Officer; Senior Vice President - Power Supply; Department Head - System Planning and Coordination; Plant Manager and others, as appropriate on at least a bi-monthly frequency.

6.5.2.6 The Corporate Nuclear Safety Section review program shall be conducted in accordance with written, approved procedures.

6.5.3 Independent Off-Site Quality Assurance Audit Program

6.5.3.1 Purpose

Audits of activities shall be performed under the cognizance of the Corporate Quality Assurance Audit (CQAA) Section. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The training and qualifications of the entire facility staff at least once per year.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety at least once per six months.
- d. The verification of compliance and implementation of the requirements of the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- e. The Emergency Plan and implementing procedures at least once per two years.
- f. The Security Plan and implementing procedures at least once per two years.
- g. Any other area of facility operation considered appropriate by the Corporate Quality Assurance Audit Section or the Senior Vice President - Power Supply.

6.5.3.2 Responsibility

The Manager - Corporate Quality Assurance Audit is charged with the overall responsibility for the corporate quality assurance audit program as follows:

- a. Selects auditors and
- b. Has access to records and personnel necessary in performing the audits.

6.5.3.3 Personnel

- a. Audit personnel will be independent of the area audited. Selection for auditing assignments is based on experience or training which establishes that their qualifications are commensurate with the complexity or special nature of the activities to be audited. In selecting auditing personnel, consideration will be given to special abilities, specialized technical training, prior pertinent experience, personal characteristics, and education.
- b. Qualified outside consultants or other individuals independent from those personnel directly involved in plant operation, but within organizations reporting to the chief operating officer, will be used to augment the audit teams when necessary.

6.5.3.4 Reports

Results of audit are approved by the Manager - Corporate Quality Assurance Audit and transmitted directly to the Company Chief Executive Officer and the Company Chief Operating Officer, as well as, to the Senior Vice President - Power Supply, Department Head - System Planning and Coordination, and others, as appropriate within 30 days after the completion of the audit.

- 6.5.3.5 The corporate quality assurance audit program shall be conducted in accordance with written, approved procedures.

6.7

Safety Limit Violation

If a safety limit is exceeded, the affected reactor shall be shut down and reactor operation shall only be resumed in accordance with the authorization within 10CFR50.36(c)(1)(i). An immediate report shall be made to the Manager - Nuclear Generation, to the Manager - Corporate Nuclear Safety, and to the Office of Inspection and Enforcement, NRC.

A complete analysis of the circumstances leading up to and resulting from the situation together with recommendations to prevent a recurrence shall be prepared by the Plant Nuclear Safety Committee. This report shall be submitted to the Manager - Nuclear Generation and the Manager - Corporate Nuclear Safety for independent review. Appropriate analyses or reports shall be submitted to the NRC by the Generation Department within 14 days of the violation.

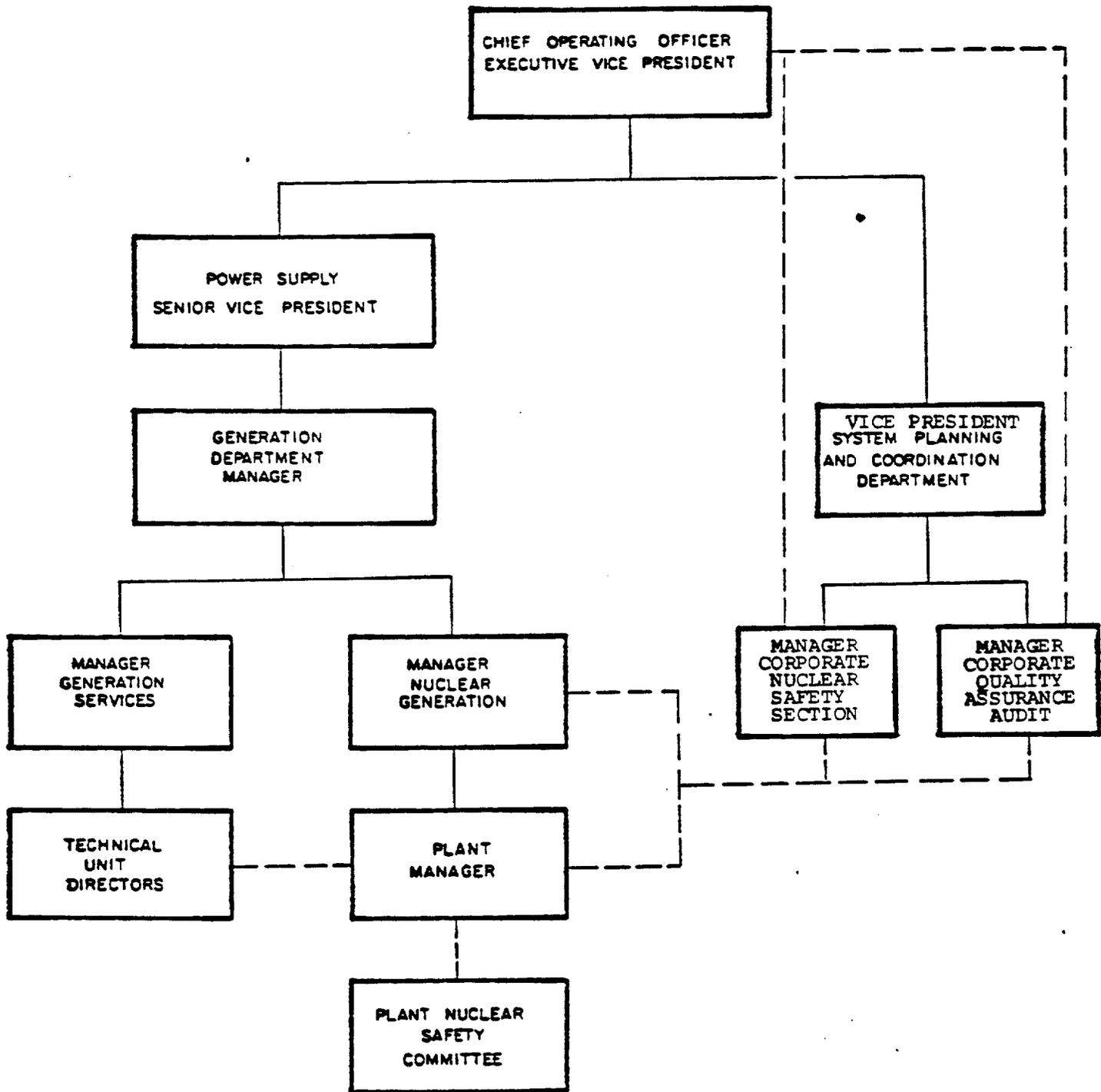
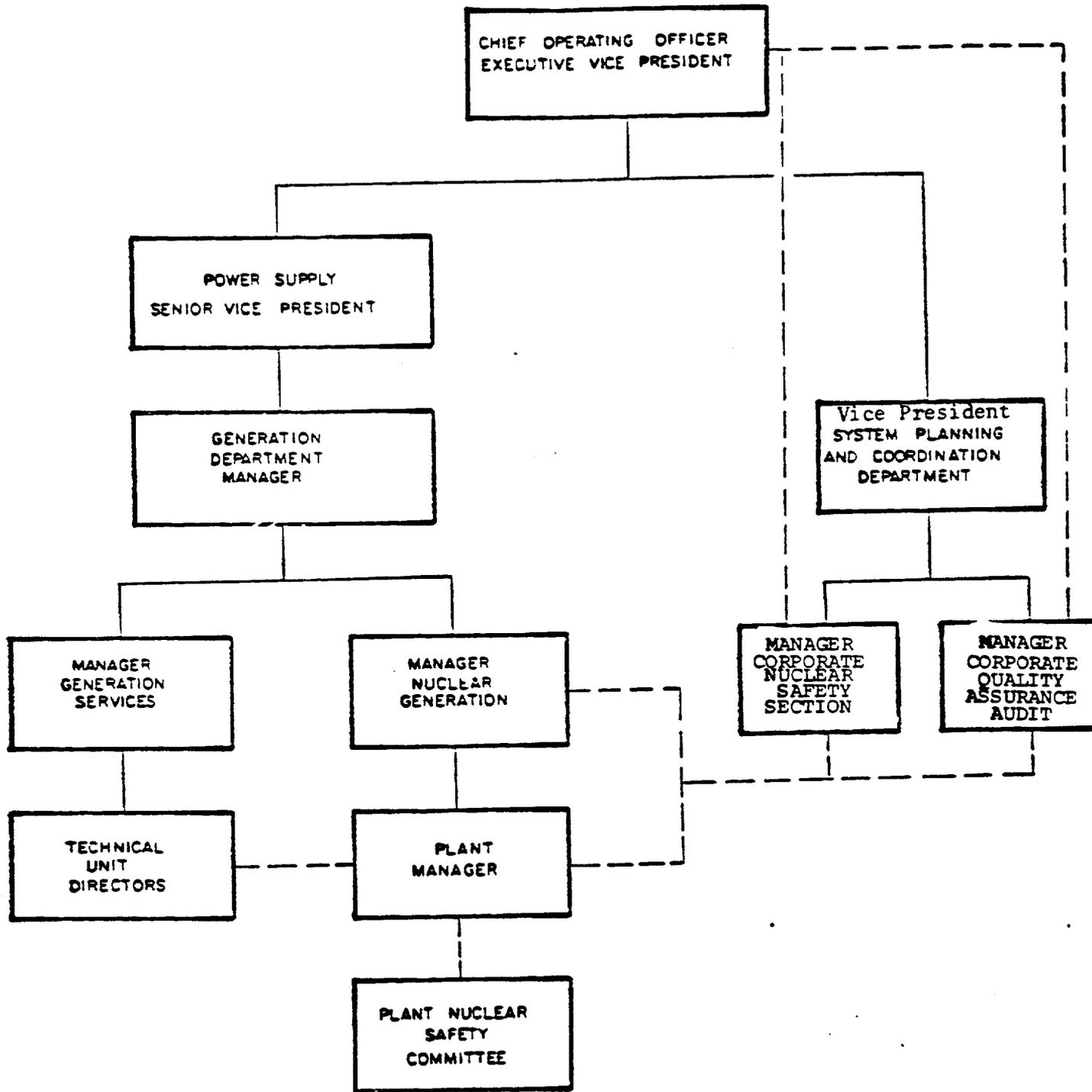


Figure 6.2-1

Amendment No. 6



MANAGEMENT ORGANIZATION CHART
FIGURE 8.2-1

ADMINISTRATIVE CONTROLS

RESPONSIBILITY

6.5.2.2 The Manager - Corporate Nuclear Safety under the Department Head - System Planning and Coordination is charged with the overall responsibility for administering the independent off site nuclear safety review program as follows:

- a. Approves selection of the person or persons to conduct off site safety reviews.
- b. Has access to the plant operating records and operating personnel in performing the independent reviews.
- c. Prepares and retains written records of reviews.
- d. Assures independent safety review is conducted on all items required by Section 6.5.2.4.
- e. Distributes reports and other records to appropriate managers.

PERSONNEL

6.5.2.3

- a. Personnel assigned responsibility for independent reviews shall be specified in technical disciplines, and shall collectively have the experience and competence required to review problems in the following areas:
 1. Nuclear power plant operations
 2. Nuclear engineering
 3. Chemistry and radiochemistry
 4. Metallurgy
 5. Instrumentation and control
 6. Radiological safety

ADMINISTRATIVE CONTROLS

- b. Recommendations and concerns will be submitted to the Manager - Nuclear Generation within 14 days of determination.
- c. A summation of Corporate Nuclear Safety Section recommendations and concerns will be submitted to the Company Chief Executive Officer; Company Chief Operating Officer; Senior Vice President - Power Supply; Department Head - System Planning and Coordination; Plant Manager and others, as appropriate on at least a bi-monthly frequency.

6.5.2.6 The Corporate Nuclear Safety Section review program shall be conducted in accordance with written, approved procedures.

6.5.3 INDEPENDENT OFF SITE QUALITY ASSURANCE AUDIT PROGRAM

PURPOSE

6.5.3.1 Audits of activities shall be performed under the cognizance of the Corporate Quality Assurance Audit (CQAA) Section. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The training and qualifications of the entire facility staff at least once per year.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety at least once per six months.
- d. The verification of compliance and implementation of the requirements of the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- e. The Emergency Plan and implementing procedures at least once per two years.
- f. The Security Plan and implementing procedures at least once per two years.
- g. Any other area of facility operation considered appropriate by the Corporate Quality Assurance Audit Section or the Senior Vice President - Power Supply.

ADMINISTRATIVE CONTROLS

RESPONSIBILITY

6.5.3.2 The Manager - Corporate Quality Assurance Audit is charged with the overall responsibility for the corporate quality assurance audit program as follows:

- a. Selects auditors and
- b. Has access to records and personnel necessary in performing the audits.

PERSONNEL

6.5.3.3

- a. Audit personnel will be independent of the area audited. Selection for auditing assignments is based on experience or training which establishes that their qualifications are commensurate with the complexity or special nature of the activities to be audited. In selecting auditing personnel, consideration will be given to special abilities, specialized technical training, prior pertinent experience, personal characteristics, and education.
- b. Qualified outside consultants or other individuals independent from those personnel directly involved in plant operation, but within organizations reporting to the chief operating officer, will be used to augment the audit teams when necessary.

REPORTS

6.5.3.4 Results of audit are approved by the Manager - Corporate Quality Assurance Audit and transmitted directly to the Company Chief Executive Officer and Company Chief Operating Officer, as well as to the Senior Vice President - Power Supply, Department Head - System Planning and Coordination and others, as appropriate within 30 days after the completion of the audit.

6.5.3.5 The corporate quality assurance audit program shall be conducted in accordance with written, approved procedures.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

Objective

This section describes the administrative controls and procedures necessary to implement the Environmental Technical Specifications.

5.1

ORGANIZATION AND REVIEW

The Plant Manager is directly responsible for the safe operation of the facility as shown in Figure 5.1-1. In all matters pertaining to the operation of the plant and to the Environmental Technical Specifications, the Plant Manager is directly responsible to the Manager of Nuclear Generation. The Environmental and Radiation Control Supervisor is directly responsible to the Plant Manager for all Environmental Technical Specifications applicable to the plant, radiological and otherwise. The Radiation Control Unit of the Generation Services Section reports to the Manager - Generation Services and functions in a staff capacity to assist in the proper implementation of the Environmental Technical Specifications.

Review of plant operations and the technical specifications shall be accomplished by the Plant Nuclear Safety Committee (PNSC) and the Corporate Nuclear Safety (CNS) Section as organizationally described in Section 6.5 of Appendix A to the facility operating license. Independent off-site QA audits of plant operations shall be performed by the Corporate Quality Assurance Audit (CQAA) Section as described in Appendix A to the facility operating license.

Review and audit functions are defined as follows:

- a. Review by PNSC and CNS of proposed changes to the Environmental Technical Specifications and the evaluated impact of the change.

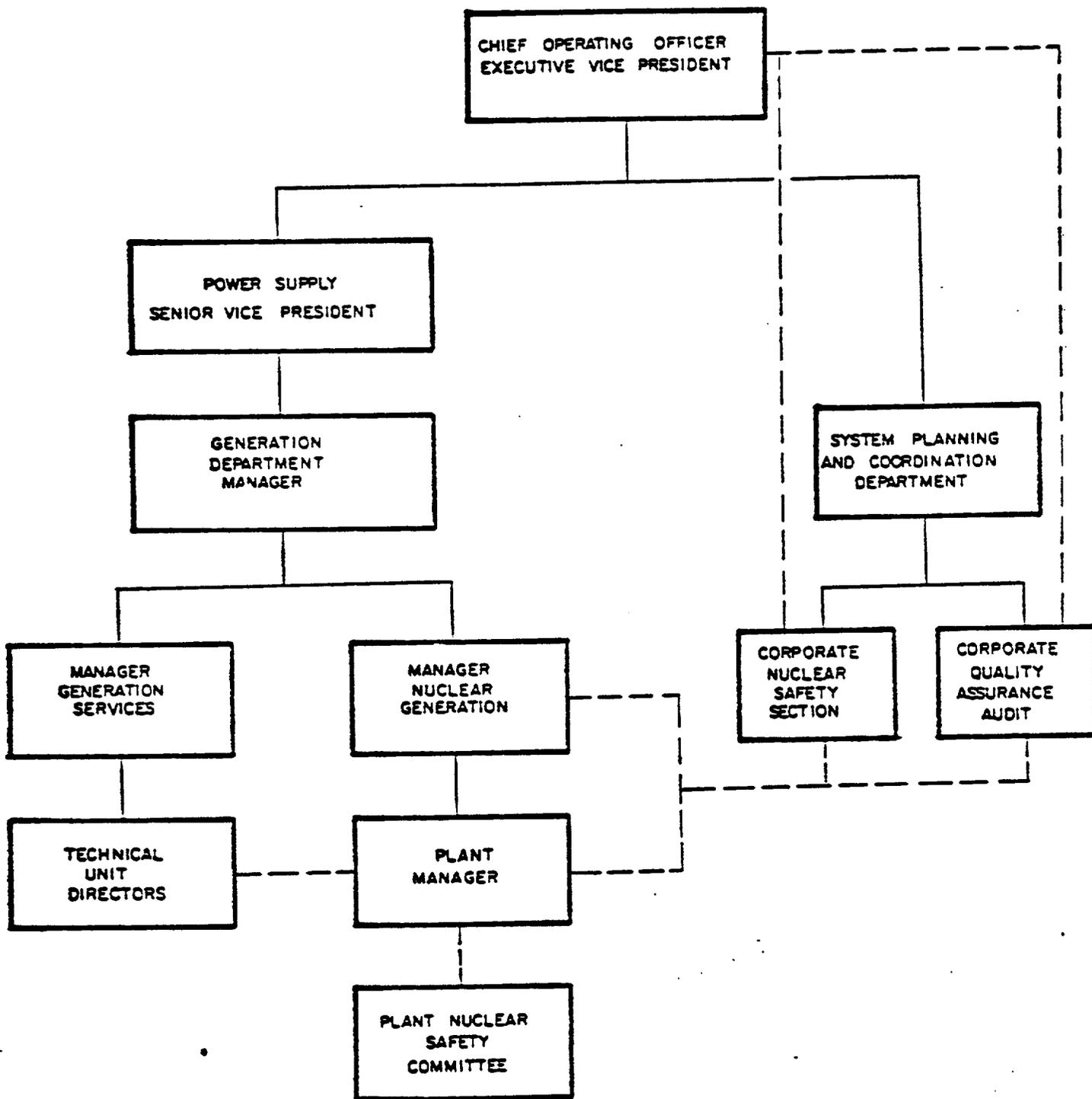


Figure 5.1-1

Amendment No. 6



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 28
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated March 21, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility License No. DPR-62 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 28, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 20, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 28

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Revise Appendix A and B Technical Specifications by removing the following pages and replacing with identically numbered revised pages:

Appendix A

6.5-4
6.5-6
6.5-7
6.5-8
6.7-1
Fig. 6.2-1

Appendix B

5-1
Fig. 5.1-1

6.5.2 Independent Off-Site Safety Review Program

Activities occurring during plant operations shall be independently reviewed as specified in succeeding paragraphs.

6.5.2.1 Purpose

The purpose of the independent off-site safety review program is to review significant plant changes, tests, and procedures; verify that reportable occurrences are promptly investigated and corrected in a manner which reduces the probability of recurrence of such events; and detect trends which may not be apparent to a day-to-day observer.

6.5.2.2 Responsibility

The Manager - Corporate Nuclear Safety under the Department Head - System Planning and Coordination is charged with the overall responsibility for administering the independent off-site nuclear safety review program as follows:

- a. Approves selection of the person or persons to conduct off-site safety reviews.
- b. Has access to the plant operating records and operating personnel in performing the independent reviews.
- c. Prepares and retains written records of reviews.
- d. Assures independent safety review is conducted on all items required by Section 6.5.2.4.
- e. Distributes reports and other records to appropriate managers.

6.5.2.3 Personnel

- a. Personnel assigned responsibility for independent reviews shall be specified in technical disciplines, and shall collectively have the experience and competence required to review problems in the following areas:
 1. Nuclear power plant operations
 2. Nuclear engineering
 3. Chemistry and radiochemistry
 4. Metallurgy
 5. Instrumentation and control
 6. Radiological safety

technical specifications or an unreviewed safety questions pursuant to 10 CFR 50.59(c). Matters of this kind shall be referred to the Corporate Nuclear Safety Section by the Plant Nuclear Safety Committee following its review, or by other functional organizational units within Carolina Power & Light Company (CP&L) prior to implementation.

- c. Changes in the technical specifications or license amendments relating to nuclear safety prior to implementation, except in those cases where the change is identical to a previously reviewed proposed change.
- d. Violations, deviations and reportable events, which require reporting to the NRC within 24 hours, and as defined in the plant technical specifications such as:
 - (1) Violations of applicable codes, regulations, order, technical specifications, license requirements or internal procedures or instructions having safety significance; and
 - (2) Significant operating abnormalities or deviations from normal or expected performance of plant safety-related structures, systems, or components.

Review of events covered under this paragraph shall include the results of any investigations made and the recommendations resulting from such investigations to prevent or reduce the probability of recurrence of the event.

- e. Any other matter involving safe operation of the nuclear power plant which the Manager - Corporate Nuclear Safety Section deem appropriate for consideration, or which is referred to the Manager - Corporate Nuclear Safety Section by the onsite operating organization or by other functional organizational units within Carolina Power & Light Company.

6.5.2.5 Follow-Up Action

Results of Corporate Nuclear Safety (CNS) Section reviews, including recommendations and concerns will be documented.

- a. Copies of the documented review will be retained in the Corporate Nuclear Safety Section files.
- b. Recommendations and concerns will be submitted to the Manager - Nuclear Generation within 14 days of determination.
- c. A summation of Corporate Nuclear Safety Section recommendations and concerns will be submitted to the Company Chief Executive Officer;

Company Chief Operating Officer; Senior Vice President - Power Supply; Department Head - System Planning and Coordination; Plant Manager and others, as appropriate on at least a bi-monthly frequency.

6.5.2.6 The Corporate Nuclear Safety Section review program shall be conducted in accordance with written, approved procedures.

6.5.3 Independent Off-Site Quality Assurance Audit Program

6.5.3.1 Purpose

Audits of activities shall be performed under the cognizance of the Corporate Quality Assurance Audit (CQAA) Section. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The training and qualifications of the entire facility staff at least once per year.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety at least once per six months.
- d. The verification of compliance and implementation of the requirements of the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- e. The Emergency Plan and implementing procedures at least once per two years.
- f. The Security Plan and implementing procedures at least once per two years.
- g. Any other area of facility operation considered appropriate by the Corporate Quality Assurance Audit Section or the Senior Vice President - Power Supply.

6.5.3.2 Responsibility

The Manager - Corporate Quality Assurance Audit is charged with the overall responsibility for the corporate quality assurance audit program as follows:

- a. Selects auditors and
- b. Has access to records and personnel necessary in performing the audits.

6.5.3.3 Personnel

- a. Audit personnel will be independent of the area audited. Selection for auditing assignments is based on experience or training which establishes that their qualifications are commensurate with the complexity or special nature of the activities to be audited. In selecting auditing personnel, consideration will be given to special abilities, specialized technical training, prior pertinent experience, personal characteristics, and education.
- b. Qualified outside consultants or other individuals independent from those personnel directly involved in plant operation, but within organizations reporting to the chief operating officer, will be used to augment the audit teams when necessary.

6.5.3.4 Reports

Results of audit are approved by the Manager - Corporate Quality Assurance Audit and transmitted directly to the Company Chief Executive Officer and the Company Chief Operating Officer, as well as, to the Senior Vice President - Power Supply, Department Head - System Planning and Coordination, and others, as appropriate within 30 days after the completion of the audit.

- 6.5.3.5** The corporate quality assurance audit program shall be conducted in accordance with written, approved procedures.

6.7

Safety Limit Violation

If a safety limit is exceeded, the affected reactor shall be shut down and reactor operation shall only be resumed in accordance with the authorization within 10CFR50.36(c)(1)(i). An immediate report shall be made to the Manager - Nuclear Generation, to the Manager - Corporate Nuclear Safety, and to the Office of Inspection and Enforcement, NRC.

A complete analysis of the circumstances leading up to and resulting from the situation together with recommendations to prevent a recurrence shall be prepared by the Plant Nuclear Safety Committee. This report shall be submitted to the Manager - Nuclear Generation and the Manager - Corporate Nuclear Safety for independent review. Appropriate analyses or reports shall be submitted to the NRC by the Generation Department within 14 days of the violation.

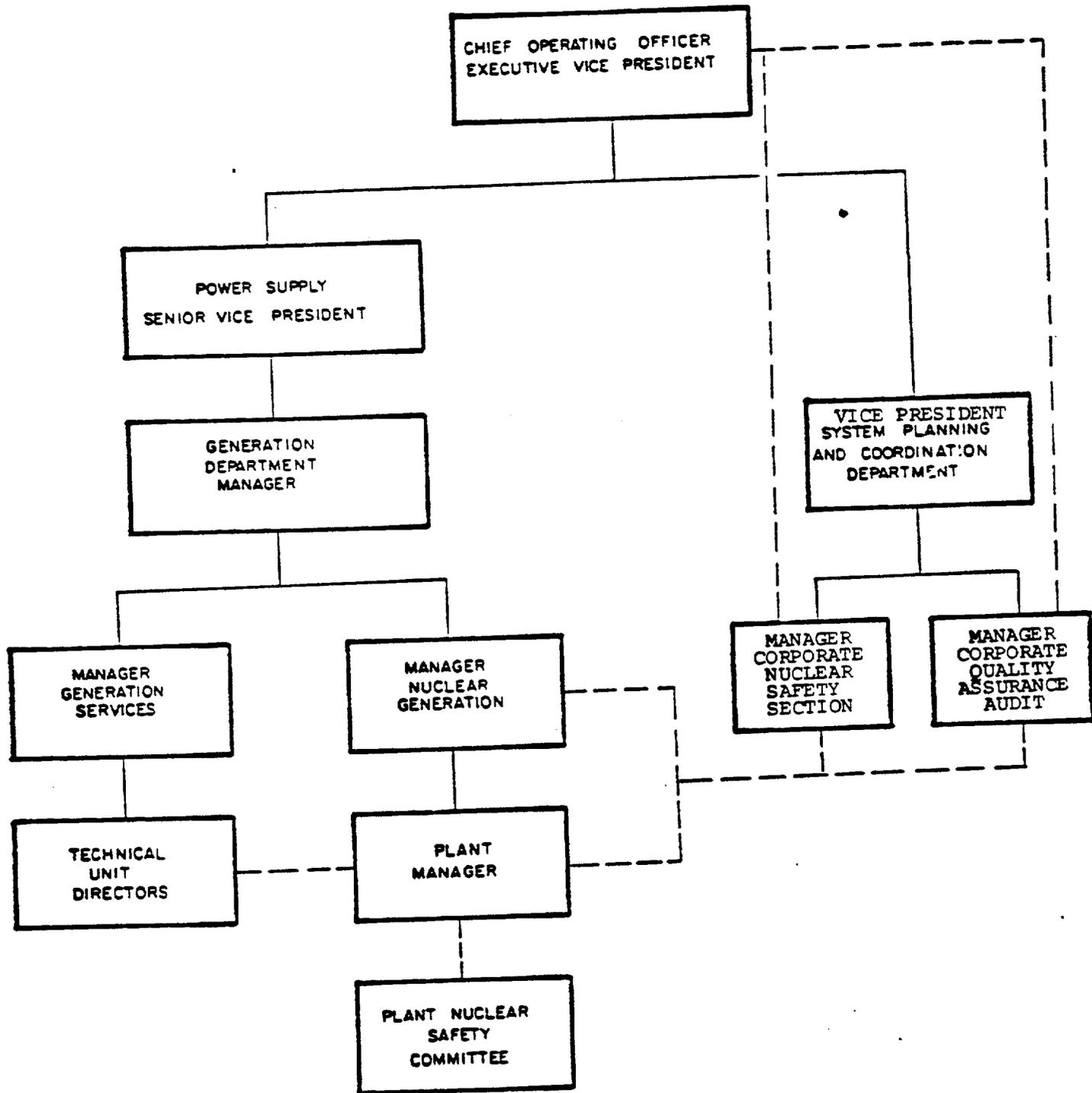


Figure 6.2-1

Amendment No. 28

Administrative Controls

Objective

This section describes the administrative controls and procedures necessary to implement the Environmental Technical Specifications.

5.1

ORGANIZATION AND REVIEW

The Plant Manager is directly responsible for the safe operation of the facility as shown in Figure 5.1-1. In all matters pertaining to the operation of the plant and to the Environmental Technical Specifications, the Plant Manager is directly responsible to the Manager of Nuclear Generation. The Environmental and Radiation Control Supervisor is directly responsible to the Plant Manager for all Environmental Technical Specifications applicable to the plant, radiological and otherwise. The Radiation Control Unit of the Generation Services Section reports to the Manager - Generation Services and functions in a staff capacity to assist in the proper implementation of the Environmental Technical Specifications.

Review of plant operations and the technical specifications shall be accomplished by the Plant Nuclear Safety Committee (PNSC) and the Corporate Nuclear Safety (CNS) Section as organizationally described in Section 6.5 of Appendix A to the facility operating license. Independent off-site QA audits of plant operations shall be performed by the Corporate Quality Assurance Audit (CQAA) Section as described in Appendix A to the facility operating license.

Review and audit functions are defined as follows:

- a. Review by PNSC and CNS of proposed changes to the Environmental Technical Specifications and the evaluated impact of the change.

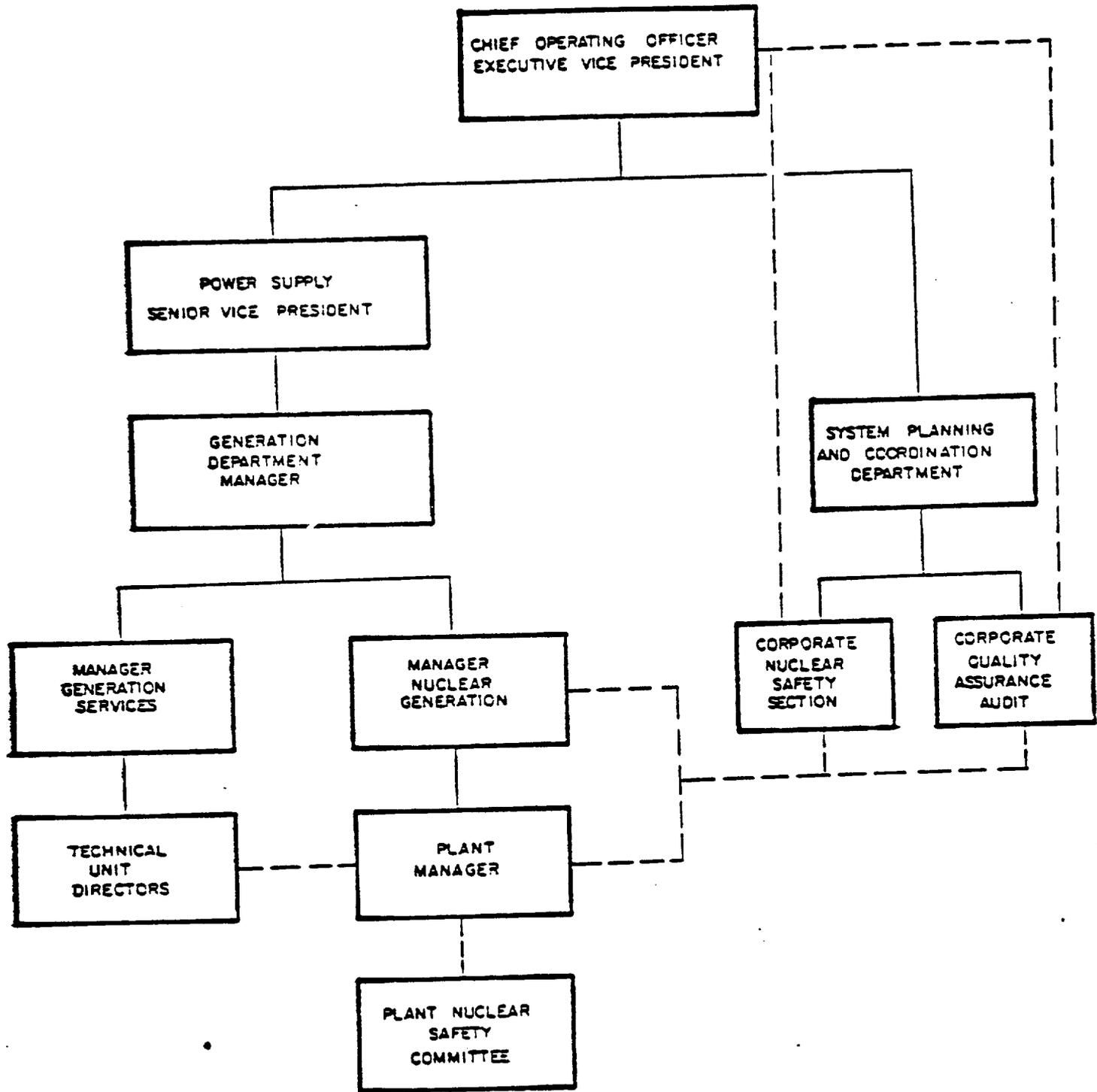


Figure 5.1-1

Amendment No. 28



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE NO. DPR-71
AND AMENDMENT NO. 28 TO FACILITY OPERATING LICENSE NO. DPR-62
CAROLINA POWER AND LIGHT COMPANY
BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 & 2
DOCKETS NOS. 50-325 AND 50-324

Introduction

By letter dated March 21, 1977, Carolina Power & Light Company (the licensee) requested changes to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-71 and DPR-62 for operation of Brunswick Steam Electric Plant, Units Nos. 1 and 2. The changes were requested to reflect a revised off-site company organization for facility management and technical support and to make editorial changes to the administrative sections of the Technical Specifications.

In the course of our review, we suggested the addition of position titles to some of the blocks shown on the new organization chart. This was suggested to clarify the relationship between the organization shown in the organization chart and the position titles used in the text of the technical specifications. We also suggested a minor revision in wording to clarify the meaning of "Operations Group," i.e. those organizations reporting to the Chief Operating Officer. The licensee has agreed with both of these modifications.

Discussion

The organizational changes consist of the following:

1. The reporting official for the Manager - Corporate Nuclear Safety Section (CNSS) and the Manager - Corporate Quality Assurance Audit Section has been changed from the Department Manager of the Technical Services Department to the Department Head - System Planning and Coordination Department.
2. Recommendations of the CNSS and audit results from the Corporate Quality Assurance Audit Section are now submitted to the Company Chief Executive Officer; Company Chief Operating Officer; Senior

Vice President - Power Supply; and the Department Head - System Planning and Coordination. Previously such recommendations and audit results were submitted to the Company President; Group Executive - Engineering, Construction and Operation Group; Department Head - Bulk Power Supply; and the Department Head - Technical Services Department.

3. The title of Vice President - Bulk Power Supply has been changed to Senior Vice President - Power Supply.
4. Supplementary personnel for audit teams will be taken, when needed, from the Operations Group, instead of the Engineering, Construction and Operation Group.
5. Reports of any safety limit violations are now the responsibility of the Generation Department, instead of the Bulk Power Supply Department.
6. The Radiation Control Unit is now assigned to the Generation Services Section instead of the Nuclear Generation Section.

Evaluation

The off-site review and audit function continues to be performed by the Corporate Nuclear Safety Section and the Corporate Quality Assurance Audit Section. These groups continue to report directly to a senior level of company management, now the Department Head - System Planning and Coordination Department which is independent from the organizational unit having direct responsibility for plant operation. This assures continuity of the already established off-site review and audit capability to verify that organizational checks and balances are functioning to assure continued safety operation and design adequacy of the plant. These two organizational groups collectively continue to contain the essential elements of a comprehensive review and audit program which is consistent with ANSI N18.7-1972, "Administrative Controls for Nuclear Power Plants". We find that the licensee's change in reporting official is acceptable.

Based on the foregoing and our review of the other changes described above, we conclude that the licensee's organizational structure for the management and technical support of the Brunswick Steam Electric Plant continues to be acceptable as revised.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 20, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-325 & 324

CAROLINA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 6 to Facility Operating License No. DPR-71 and Amendment No. 28 to Facility Operating License No. DPR-62, issued to Carolina Power and Light Company (the licensee), which revised Technical Specifications for operation of Brunswick Steam Electric Plant, Units Nos. 1 & 2 (the facility) located in Brunswick County, North Carolina. The amendments are effective as of the date of issuance.

The amendments revise the off-site corporate organization for facility management and technical support, and make several editorial changes to the administrative sections of the Technical Specifications to reflect the revised organization.

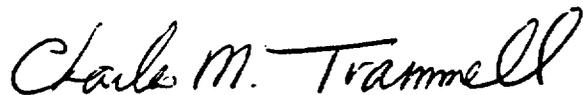
The application for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendment dated March 21, 1977, (2) Amendment No. 6 to License No. DPR-71, (3) Amendment No. 28 to License No. DPR-62, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Southport-Brunswick County Library, 109 West Moore Street Southport, North Carolina 28461. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 20th day of June 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



Charles M. Trammell, Acting Chief
Operating Reactors Branch #1
Division of Operating Reactors