

November 13, 1995

Mr. Oliver D. Kingsley, Jr.
President, TVA Nuclear and
Chief Nuclear Officer
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

SUBJECT: ISSUANCE OF TECHNICAL SPECIFICATION AMENDMENTS FOR THE BROWNS FERRY
NUCLEAR PLANT UNIT 3 (TAC NO. M93657)

Dear Mr. Kingsley:

The Commission has issued the enclosed Amendment No. 202 to Facility Operating Licenses No. DPR-68 for the Browns Ferry Nuclear Plant (BFN), Unit 3. This amendment is in response to your application dated September 13, 1995, regarding removal of the requirement to perform daily instrument checks for the Steam Supply Low Pressure and Turbine Exhaust Diaphragm High Pressure for both the Reactor Core Isolation Cooling and High Pressure Coolant Injection systems, and correction of a typographical error.

A copy of the NRC's Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Joseph F. Williams, Project Manager
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-296

Enclosures: 1. Amendment No. 202 to
License No. DPR-68
2. Safety Evaluation

cc w/enclosures: See next page

Distribution w/enclosure

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-296

BROWNS FERRY NUCLEAR PLANT, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 202
License No. DPR-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated September 13, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

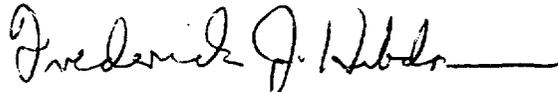
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-68 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 202, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 13, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 202

FACILITY OPERATING LICENSE NO. DPR-68

DOCKET NO. 50-296

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. *Overleaf page is provided to maintain document completeness.

REMOVE

3.2/4.2-45
3.2/4.2-46
3.2/4.2-49
3.2/4.2-49a

INSERT

3.2/4.2-45
3.2/4.2-46
3.2/4.2-49
3.2/4.2-49a*

TABLE 4.2.B (Cont'd)
SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE OR CONTROL THE CSCS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration</u>	<u>Instrument Check</u>
Instrument Channel - RHR Pump Discharge Pressure	(1)	once/3 months	none
Instrument Channel - Core Spray Pump Discharge Pressure	(1)	once/3 months	none
Core Spray Sparger to RPV d/p	(1)	once/3 months	once/day
Trip System Bus Power Monitor	once/operating Cycle	N/A	none
Instrument Channel - Condensate Header Level (LS-73-56A, B)	(1)	once/3 months	none
Instrument Channel - Suppression Chamber High Level	(1)	once/3 months	none
Instrument Channel - Reactor High Water Level (LIS-3-208A-D)	(1)(28)	once/18 months (29)	once/day
Instrument Channel - RCIC Turbine Steam Line High Flow	(1)(28)	once/18 months (29)	none
Instrument Channel - RCIC Steam Supply Low Pressure	once/31 days	once/18 months	none
Instrument Channel - RCIC Turbine Exhaust Diaphragm High Pressure	once/31 days	once/18 months	none
RCIC Steam Line Space Torus Area High Temperature	(1)	once/3 months	none
RCIC Steam Line Space RCIC Pump Room Area High Temperature	(1)	once/3 months	none
HPCI Steam Line Space Torus Area High Temperature	(1)	once/3 months	none
HPCI Steam Line Space HPCI Pump Room Area High Temperature	(1)	once/3 months	none

BFN
Unit 3

3.2/4.2-45

AMENDMENT NO. 202

TABLE 4.2.B (Cont'd)
 SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE OR CONTROL THE CSCS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration</u>	<u>Instrument Check</u>
Instrument Channel - HPCI Turbine Steam Line High Flow	(1)(28)	once/18 months (29)	none
Instrument Channel - HPCI Steam Supply Low Pressure	once/31 days	once/18 months	none
Instrument Channel - HPCI Turbine Exhaust Diaphragm High Pressure	once/31 days	once/18 months	none
Core Spray System Logic	once/18 months	(6)	N/A
RCIC System (Initiating) Logic	once/18 months	N/A	N/A
RCIC System (Isolation) Logic	once/18 months	(6)	N/A
HPCI System (Initiating) Logic	once/18 months	(6)	N/A
HPCI System (Isolation) Logic	once/18 months	(6)	N/A
ADS Logic	once/18 months	(6)	N/A
LPCI (Initiating) Logic	once/18 months	(6)	N/A
LPCI (Containment Spray) Logic	once/18 months	(6)	N/A

BFN
Unit 3

3.2/4.2-46

AMENDMENT NO. 202

TABLE 4.2.C
SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE ROD BLOCKS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration (17)</u>	<u>Instrument Check</u>
APRM Upscale (Flow Bias)	(1) (13)	once/3 months	once/day (8)
APRM Upscale (Startup Mode)	(1) (13)	once/3 months	once/day (8)
APRM Downscale	(1) (13)	once/3 months	once/day (8)
APRM Inoperative	(1) (13)	N/A	once/day (8)
RBM Upscale (Flow Bias)	(1) (13)	once/6 months	once/day (8)
RBM Downscale	(1) (13)	once/6 months	once/day (8)
RBM Inoperative	(1) (13)	N/A	once/day (8)
IRM Upscale	(1)(2) (13)	once/3 months	once/day (8)
IRM Downscale	(1)(2) (13)	once/3 months	once/day (8)
IRM Detector Not in Startup Position	(2) (once operating cycle)	once/operating cycle (12)	N/A
IRM Inoperative	(1)(2) (13)	N/A	N/A
SRM Upscale	(1)(2) (13)	once/3 months	once/day (8)
SRM Downscale	(1)(2) (13)	once/3 months	once/day (8)
SRM Detector Not in Startup Position	(2) (once/operating cycle)	once/operating cycle (12)	N/A
SRM Inoperative	(1)(2) (13)	N/A	N/A
Flow Bias Comparator	(1)(15)	once/operating cycle (20)	N/A
Flow Bias Upscale	(1)(15)	once/3 months	N/A
Rod Block Logic	(16)	N/A	N/A
West Scram Discharge Tank Water Level High (LS-85-45L)	once/quarter	once/operating cycle	N/A
East Scram Discharge Tank Water Level High (LS-85-45M)	once/quarter	once/operating cycle	N/A

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Unit 3

3.2/4.2-49

AMENDMENT NO. 202

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 202 TO FACILITY OPERATING LICENSE NO. DPR-68
TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT, UNIT 3
DOCKET NO. 50-296

1.0 INTRODUCTION

By letter dated September 13, 1995, the Tennessee Valley Authority (the licensee) requested changes to the Technical Specifications (TS) for the Browns Ferry Nuclear Plant (BFN) Unit 3. The proposed changes would revise TS Table 4.2.B by removing the requirement to perform daily instrument checks for the Steam Supply Low Pressure and Turbine Exhaust Diaphragm high Pressure for both the Reactor Core Isolation Cooling (RCIC) and High Pressure Coolant Injection (HPCI) systems, and corrects an editorial error.

2.0 BACKGROUND

The proposed change to TS Table 4.2.B deletes the requirement for instrument checks for instrumentation used to trip and isolate the RCIC and HPCI systems on either low steam supply pressure or high turbine exhaust pressure. These instrument channels have no method of indication; therefore, the performance of instrument checks is not appropriate.

The requirement for a daily instrument check for these instrument channels had been added in error by TS-243 (submitted June 13, 1988). That TS change added the instrument channels for the RCIC and HPCI steam supply low pressure and turbine exhaust diaphragm high pressure to TS Tables 3.2.B and 4.2.B for BFN Units 1, 2, and 3. The proposed surveillance requirements included daily instrument checks. TS-243 was issued on September 23, 1988.

The change to Table 4.2.C corrects a typographical error made during the retyping and reformatting of the TS that occurred in 1987. The source range monitor (SRM) Upscale functional test incorrectly references note 15 (Flow Bias Comparator). The correct reference should be note 13. This note is correctly referenced in previous revisions to Unit 3 TS Table 4.2.C and in the current TS for Units 1 and 2.

3.0 EVALUATION

The proposed change to Table 4.2.B corrects an error introduced during a previous TS change. That TS change added the requirement to perform

ENCLOSURE 2

instrument checks for pressure switches. This error has been previously corrected for Unit 2. As defined by TS, an instrument check is a qualitative determination of acceptable Operability by observation of instrument behavior during operation. The pressure switches associated with these RCIC and HPCI instrument channels have no indication function; therefore, an instrument check is not appropriate. These instruments are functionally tested once per 31 days and calibration tests are performed once per 18 months as required by TS surveillance requirements. This required testing verifies the operability of these instrument channels. The staff agrees an instrument check would serve no purpose for these instruments. Therefore, the proposed change is acceptable.

The change to Table 4.2.C corrects a typographical error that was introduced during earlier retyping and reformatting of the Unit 3 TS. This change correctly references the functional test of the SRM Upscale function to note 13 instead of note 15. The staff finds the proposed change is appropriate and acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Alabama State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (60 FR 52935). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based upon the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: George F. Wunder

Dated: November 13, 1995

Mr. Oliver D. Kingsley, Jr.
Tennessee Valley Authority

BROWNS FERRY NUCLEAR PLANT

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