

November 25, 1991

Docket No. 50-260

Mr. Dan A. Nauman
Senior Vice President, Nuclear Power
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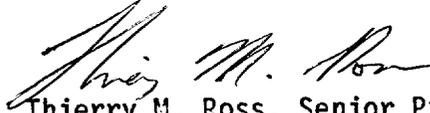
Dear Mr. Nauman:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M81052) (TS 296)

The Commission has issued the enclosed Amendment No. 201, to Facility Operating License No. DPR-52 for the Browns Ferry Nuclear Plant (BFNP), Unit 2. This license amendment is in response to an application by the Tennessee Valley Authority (TVA) dated July 11, 1991. The NRC staff reviewed and approved TVA's request to correct instrument identification numbers and other administrative type errors contained in Tables 3.2.B and 3.11.A of the BFNP, Unit 2 Technical Specifications.

A copy of the staff's Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register publication.

Sincerely,



Thierry M. Ross, Senior Project Manager
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 201 to License No. DPR-52
- 2. Safety Evaluation

cc w/enclosures:
See next page

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PDR

OP-1

OFC	: PDII-4/LA	PDII-4/PM	PDII-4/PM	OGC	PDII-4/D
NAME	: MSanders	: TRoss	: as TR	: JWilliams	: FHebbon
DATE	: 11/15/91	: 11/15/91	: 11/15/91	: 11/21/91	: 11/25/91

Mr. Dan A. Nauman

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AMENDMENT NO. 201 FOR BROWNS FERRY UNIT 2 - DOCKET NO. 50-260
DATED: November 25, 1991

Docket File

NRC PDR

Local PDR

BFN Reading File

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-260

BROWNS FERRY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 201
License No. DPR-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated July 11, 1991 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 201, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 25, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 201

FACILITY OPERATING LICENSE NO. DPR-52

DOCKET NO. 50-260

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. * Denotes overleaf page.

<u>REMOVE</u>	<u>INSERT</u>
3.2/4.2-18	3.2/4.2-18
3.2/4.2-19	3.2/4.2-19*
3.11/4.11-13	3.11/4.11-13*
3.11/4.11-14	3.11/4.11-14
3.11/4.11-15	3.11/4.11-15
3.11/4.11-16	3.11/3.11-16

TABLE 3.2.B (Continued)

Minimum No. Operable Per Trip Sys(1)	Function	Trip Level Setting	Action	Remarks
1	HPCI Trip System bus power monitor	N/A	C	1. Monitors availability of power to logic systems.
1	RCIC Trip System bus power monitor	N/A	C	1. Monitors availability of power to logic systems.
1(2)	Instrument Channel - Condensate Header Low Level (LS-73-56A & B)	\geq Elev. 551'	A	1. Below trip setting will open HPCI suction valves to the suppression chamber.
1(2)	Instrument Channel - Suppression Chamber High Level	\leq 7" above instrument zero	A	1. Above trip setting will open HPCI suction valves to the suppression chamber.
2(2)	Instrument Channel - Reactor High Water Level (LIS-3-208A and LIS-3-208C)	\leq 583" above vessel zero	A	1. Above trip setting trips RCIC turbine.
1	Instrument Channel - RCIC Turbine Steam Line High Flow (PDIS-71-1A and 1B)	\leq 450" H ₂ O (7)	A	1. Above trip setting isolates RCIC system and trips RCIC turbine.
3(2)	Instrument Channel - RCIC Steam Supply Pressure - Low (PS 71-1A-D)	\geq 50 psig	A	1. Below trip setting isolates RCIC system and trips RCIC turbine.
3(2)	Instrument Channel - RCIC Turbine Exhaust Diaphragm Pressure - High (PS 71-11A-D)	\leq 20 psig	A	1. Above trip setting isolates RCIC system and trips RCIC turbine.

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Unit 2

3.2/4.2-18

Amendment 201

TABLE 3.2.B (Continued)

Minimum No. Operable Per Trip Sys(1)	Function	Trip Level Setting	Action	Remarks
2(2)	Instrument Channel - Reactor High Water Level (LIS-3-2088 and LIS-3-2080)	≤583" above vessel zero.	A	1. Above trip setting trips HPCI turbine.
1	Instrument Channel - HPCI Turbine Steam Line High Flow (PDIS-73-1A and 1B)	≤90 psi (7)	A	1. Above trip setting isolates HPCI system and trips HPCI turbine.
3(2)	Instrument Channel - HPCI Steam Supply Pressure - Low (PS 73-1A-D)	≥100 psig	A	1. Below trip setting isolates HPCI system and trips HPCI turbine.
3(2)	Instrument Channel - HPCI Turbine Exhaust Diaphragm (PS 73-20A-D)	≤20 psig	A	1. Above trip setting isolates HPCI system and trips HPCI turbine.
1	Core Spray System Logic	N/A	B	1. Includes testing auto initiation inhibit to Core Spray Systems in other units.
1	RCIC System (Initiating) Logic	N/A	B	1. Includes Group 7 valves. Refer to Table 3.7.A for list of valves.
1	RCIC System (Isolation) Logic	N/A	B	1. Includes Group 5 valves. Refer to Table 3.7.A for list of valves.
1 (16)	ADS Logic	N/A	A	
1	RHR (LPCI) System (Initiation)	N/A	B	

BFN
Unit 2

3.2/4.2-19

AMENDMENT NO. 187

3.11/4.11 FIRE PROTECTION SYSTEMS

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.11.G FIRE-RATED ASSEMBLIES
(Cont'd)

4.11.G FIRE-RATED ASSEMBLIES
(Cont'd)

2. Each of the required fire doors shall be verified OPERABLE by inspecting the automatic hold-open, release, and closing mechanisms and latches at least semiannually and by verifying:
 - a. The OPERABILITY of the fire door supervision system for each electrically supervised fire door by performing a CHANNEL FUNCTIONAL TEST at least monthly.
 - b. That each locked-closed fire door is verified closed at least weekly.
 - c. That doors with automatic hold-open and release mechanisms are free of obstructions at least daily and perform a FUNCTIONAL TEST of these mechanisms at least once per 18 months.
 - d. That each unlocked normally closed fire door without electrical supervision is verified closed at least daily.

3.11.H Open Flames, Welding, and Burning in the Cable Spreading Room

There shall be no use of open flame, welding, or burning in the cable spreading room unless the reactor is in the COLD SHUTDOWN CONDITION.

TABLE 3.11.A

FIRE DETECTION INSTRUMENTATION

<u>Instrument Location</u> <u>Building-Elevation</u>	<u>LOCAL</u> <u>PANEL</u>	<u>Area Protected/</u> <u>Equipment</u>	<u>Detector</u> <u>Type</u>	<u>Function</u>
1. Reactor - 519	None	HPCI	Heat	Actuate Fixed Spray
2. Reactor - 519	None	RCIC	Heat	Annunciation
3. Reactor - 519/541	2-25-313	RHR	Smoke & Heat	Actuate Preaction System
4. Reactor - 565	2-25-286	General Area	Smoke & Heat	Actuate Preaction System
5. Reactor - 593	2-25-287	General Area	Smoke & Heat	Actuate Preaction System
6. Reactor - 621	2-25-303	General Area	Smoke & Heat	Actuate Preaction System
7. Reactor - 639	2-25-333	Recirculation MG Sets	Heat	Actuate AFFF System
8. Diesel Generator Units 1 and 2 - 565	None	Diesel Generator Rooms and Fuel Oil Transfer Room	Heat	Actuate CO ₂ System
9. Diesel Generator Units 1 and 2 - 565	1-25-331	Pipe and Electrical Tunnel	Smoke	Actuate Preaction System
10. Diesel Generator Units 1 and 2 - 583	1-25-331	Diesel Auxiliary Board Rooms A & B	Smoke	Door Release
11. Diesel Generator Units 1 and 2 - 583	None	Aux BD A Compt 7 Aux BD B Compt 10	Smoke	Annunciation
12. Diesel Generator Units 1 and 2 - 583	None	Diesel Auxiliary Board Rooms A & B	Heat	Actuate CO ₂ System
13. Diesel Generator Unit 3 - 565	None	Diesel Generator Rooms and Fuel Oil Pump Room	Heat	Actuate CO ₂ System
14. Diesel Generator Building Unit 3 - 565	3-25-332	Pipe and Electrical Tunnel	Smoke	Actuate Preaction System
15. Diesel Generator Building Unit 3 - 565	3-25-295	Shutdown Board Rooms 3EB & 3ED and Bus Tie Room	Smoke	Annunciation
16. Diesel Generator Building Unit 3 - 565	3-25-289	Pipe and Electrical Tunnel	Heat	Actuate Fixed Spray

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Unit 2

3.11/4.11-14

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TABLE 3.11.A (Cont'd)

FIRE DETECTION INSTRUMENTATION

<u>Instrument Location</u> <u>Building-Elevation</u>	<u>LOCAL</u> <u>PANEL</u>	<u>Area Protected/</u> <u>Equipment</u>	<u>Detector</u> <u>Type</u>	<u>Function</u>
17. Diesel Generator Building Unit 3 - 565	3-25-332	Pipe and Electrical Tunnel	Heat and Smoke	Actuate Fixed Spray
18. Diesel Generator Building Unit 3 - 583	None	Diesel Auxiliary Board Rooms 3EA & 3EB	Heat	Actuate CO ₂ System
19. Diesel Generator Building Unit 3 - 583	3-25-305	Shutdown Board Rooms 3EA, 3EB, 3EC, & 3ED; Bus Tie Room; and Diesel Auxiliary Board Rooms	Smoke	Door Release
20. Diesel Generator Building Unit 3 - 583	None	Diesel Auxiliary Board Rooms 3EA & 3EB	Smoke	Annunciation
21. Diesel Generator Building Unit 3 - 583	None	Mechanical Equipment Rooms A & B	Duct	Actuate Damper
22. Control Bay - 593	1-25-325	Instrument Shop, MG Set Rooms, Battery Room 1, and Battery Board Room 1	Smoke	Annunciation
23. Control Bay - 593	None	Auxiliary Instrument Room 1	Smoke	Annunciation
24. Control Bay - 593	None	Unit 1 and 2 Computer Room	Smoke	Annunciation
25. Control Bay - 593	2-25-326	Communications Battery/Board Room, Communications Room, MG Sets, Battery Board Room 2, and Battery Room 2	Smoke	Annunciation
26. Control Bay - 593	None	Auxiliary Instrument Room 2	Smoke	Annunciation
27. Control Bay - 593	None	Shutdown Board Room B	Duct	Actuate Damper
28. Control Bay - 593	None	Shutdown Board Room D	Duct	Annunciation
29. Control Bay - 593	None	Auxiliary Instrument Room 3	Smoke	Annunciation
30. Control Bay - 593	None	Unit 3 Computer Room	Smoke	Annunciation
31. Control Bay - 593	3-25-327	MG Set Rooms, Battery Room 3, and Battery Board Room 3	Smoke	Annunciation

BFN
Unit 2

3.11/4.11-15

Amendment 201

TABLE 3.11.A (Cont'd)

FIRE DETECTION INSTRUMENTATION

	<u>Instrument Location</u> <u>Building-Elevation</u>	<u>LOCAL</u> <u>PANEL</u>	<u>Area Protected/</u> <u>Equipment</u>	<u>Detector</u> <u>Type</u>	<u>Function</u>
32.	Control Bay - 606	1-25-323	Spreading Room A	Heat and Smoke	Actuate Preaction System
33.	Control Bay - 606	3-25-324	Spreading Room B	Heat and Smoke	Actuate Preaction System
34.	Control Bay - 606, 617	1-25-328	Mechanical Equipment Room, Locker Room, Toilet, Instrument Calibration Room, Shift Engineer Office, and Kitchen,	Heat and Smoke	Actuate Preaction System
35.	Control Bay - 617	None	Unit 1 Control Room	Smoke	Annunciation
36.	Control Bay - 617	None	Shutdown Board Room A	Duct	Actuate Damper
37.	Control Bay - 617	None	Shutdown Board Room C	Duct	Annunciation
38.	Control Bay - 617	None	Unit 2 Control Room	Smoke	Annunciation
39.	Control Bay - 617	None	Relay Room	Smoke	Annunciation
40.	Control Bay - 617	None	Unit 3 Control Room	Smoke	Annunciation
41.	Control Bay - 617	3-25-329	NRC Room, TSO Operating Room, Locker Room, Toilet and Mechanical Room	Heat and Smoke	Actuate Preaction System
42.	Control Bay - 621	None	Shutdown Board Room E	Duct	Actuate Damper
43.	Control Bay - 621	None	Shutdown Board Room F	Duct	Actuate Damper
44.	Turbine - 565	0-25-297	Cable Tunnel to Intake Pumping Station	Smoke	Annunciation
45.	Turbine - 586	1-25-283	Cable Tray Zones A, B, & C	Heat	Actuate Fixed Spray
46.	Turbine - 586	1-25-334	Cable Tray Zones A, B, C, & D	Heat and Smoke	Actuate Fixed Spray
47.	Turbine - 586	3-25-293	South Wall	Smoke	Annunciation
48.	Intake Pumping Station	0-25-296	Intake Pumping Station	Smoke	Actuate Preaction System

BPN
Unit 2

3.11/4.11-16

Amendment 201



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ENCLOSURE 2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-52

TENNESSEE VALLEY AUTHORITY

BROWNS FERRY NUCLEAR PLANT, UNIT 2

DOCKET NO. 50-260

1.0 INTRODUCTION

By letter dated July 11, 1991, the Tennessee Valley Authority (TVA) submitted a request for changes to the Browns Ferry Nuclear Plant (BFNP), Unit 2, Technical Specifications (TS). The requested changes proposed to revise TS Tables 3.2.B and 3.11.A in order to correct certain instrument and panel identification numbers and other editorial errors.

2.0 EVALUATION

TVA has proposed to make a number of editorial changes to Tables 3.2.B and 3.11.A of the BFNP, Unit 2 TS. They are intended to enhance the usability of the TS by correcting miscellaneous editorial errors primarily associated with the instrument, equipment, and area identification and/or descriptives. The following is a list of the proposed changes:

1. Table 3.2.B, "Condensate Header Low Level" - Revise switch identification numbers from "LS-73-55A and B" to "LS-73-56A and B" to correctly identify actual instrumentation.
2. Table 3.11.A, Items 3, 4, 5 and 6 - Revise detector type from "Smoke" only to "Smoke and Heat" to correctly reflect actual detector type being utilized.
3. Table 3.11.A, Item 4 - Revise local panel identification number from "2-25-291" to "2-25-286" to correctly identify the proper panel.
4. Table 3.11.A, Item 21 - Add the designation of "A and B" to "Mechanical Engineering Rooms" to more clearly specify the particular rooms as they are labeled.
5. Table 3.11.A, Item 25 - Smoke detectors for the Communications Battery/Board Room, Motor Generator Sets, Battery Board Room 2 and Battery Room 2 on elevation 593 are associated with instrument panel "2-25-326." This panel identification number would be added for completeness.

6. Table 3.11.A, Item 41 - Revise detector types associated with panel 3-25-329 from "Heat or Smoke" to "Heat and Smoke" since both types of detectors are indeed present, and
7. Table 3.11.A, Item 46 - Expand the areas being protected by panel 1-25-334 from "Cable Tray Zones A, B and C" to "Cable Tray Zones A, B, C, and D" in order to accurately reflect the actual fire protective configuration.

The NRC staff has reviewed TVA's license amendment application to revise TS Tables 3.2.B and 3.11.A. As a result of this review, the staff concludes that TVA's proposed TS changes are indeed editorial in nature and do not effect the safety function or normal operation of related equipment. Therefore, the licensee amendment application of July 11, 1991 is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Alabama State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 41586). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Ross

Date: November 25, 1991