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October 29, 2001

U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Attention: Document Control Desk

Subject: Oconee Nuclear Station Docket Numbers 50-269, 270, and 287 Technical Specification Bases (TSB) Change

Please find attached revisions to TSB 3.7.11 Spent Fuel Pool (SFP) Water Level, which were implemented on October 17, 2001. The changes revise the Bases to clarify that Oconee's analysis assumes the top of the irradiated fuel assemblies as being the top of the fuel pins, for SFP level measurement. Also, clarification is provided regarding shielding provided by SFP water.

Attachment 1 contains the new TSB pages and Attachment 2 contains the markup version of the Bases pages.

If any additional information is needed, please contact Larry E. Nicholson, at (864-885-3292).

Very truly yours,

W. R. McCollum, Jr., Vice President Oconee Nuclear Site

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U. S. Nuclear Regulatory Commission October 29, 2001 Page 2

cc: Mr. L. N. Olshan Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Washington, D. C. 20555

> Mr. L. A. Reyes, Regional Administrator U. S. Nuclear Regulatory Commission - Region II Atlanta Federal Center 61 Forsyth St., SW, Suite 23T85 Atlanta, Georgia 30303

M. C. Shannon Senior Resident Inspector Oconee Nuclear Station

Virgil R. Autry, Director Division of Radioactive Waste Management Bureau of Land and Waste Management Department of Health & Environmental Control 2600 Bull Street Columbia, SC 29201 Attachment 1

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B 3.7 PLANT SYSTEMS

B 3.7.11 Spent Fuel Pool Water Level

BASES BACKGROUND The minimum water level in the Spent Fuel Pool is consistent with the assumption of iodine decontamination factors following a fuel handling or cask drop accident. The water also provides shielding during the movement of spent fuel. A general description of the Spent Fuel Pool design is given in the UFSAR, Section 9.1.2, Reference 1. The Spent Fuel Pool Cooling and Cleanup System is given in the UFSAR, Section 9.1.3 (Ref. 2). The assumptions of the fuel handling accident or cask drop are given in the UFSAR, Section 15.11.2 (Ref. 3). **APPLICABLE** During movement of irradiated fuel assemblies or crane operations with SAFETY ANALYSES loads in the Spent Fuel Pool, the water level in the pool is an initial condition design parameter in the analysis of the fuel handling accident and cask drop accidents in the fuel pool. A minimum water level of 23 ft (Regulatory Position C.1.c of Ref. 4) allows a decontamination factor (DF) of 100 (Regulatory Position C.1.g of Ref. 4) to be used in the accident analysis for iodine. This relates to the assumption that 99% of the total iodine released from the pellet to cladding gap of all the damaged fuel assembly(ies) rods is retained by the Spent Fuel Pool water. The fuel pellet to cladding gap is assumed to contain 10% of the total fuel rod iodine inventory (Ref. 4). The fuel handling accident and cask drop accident analysis in the Spent Fuel Pool is described in Reference 3. Since the minimum water level of 21.34 feet is less than 23 feet, the assumed iodine DF must be less than 100, according to Ref. 4, and calculated with comparable conservatism. Oconee's analysis assumes the top of the irradiated fuel assemblies as the top of the fuel pins (Refs. 4 and 8). An experimental test program described in WCAP-7828 (Ref. 6) evaluated the extent of removal of iodine released from a damaged irradiated fuel assembly. Using the analytical results from the test program described in WCAP-7828, with a water depth of 21.34 feet, a comparable DF of 89 was determined. With a minimum water level of

OCONEE UNITS 1, 2, & 3

BASES		
APPLICABLE SAFETY ANALYSES (continued)	21.34 ft, and a minimum decay time of 72 hours prior to fuel handling, the analysis and test programs demonstrate that the iodine release due to a postulated fuel handling or cask drop accident is adequately captured by the water, and offsite doses are maintained within allowable limits (Ref. 7).	
	The Spent Fuel Pool water level satisfies Criterion 2 and 3 of 10 CFR 50.36 (Ref. 7).	
LCO	The specified water level preserves the assumptions of the fuel handling and cask drop accident analyses (Ref. 3). As such, it is the minimum required for fuel storage and movement within the Spent Fuel Pool or movement of the cask over the Spent Fuel Pool.	
APPLICABILITY	This LCO applies during movement of irradiated fuel assemblies in the Spent Fuel Pool or movement of the cask over the Spent Fuel Pool since the potential for a release of fission products exists.	
ACTIONS	Required Actions A.1 and A.2 are modified by a Note indicating that LCO 3.0.3 does not apply.	
	If moving irradiated fuel assemblies or a cask while in MODE 5 or 6, LCO 3.0.3 would not specify any action. If moving irradiated fuel assemblies or a cask while in MODES 1, 2, 3, and 4, the fuel or cask movement is independent of reactor operations. Therefore, in either case, inability to suspend movement of irradiated fuel assemblies or a cask is not sufficient reason to require a reactor shutdown.	
	<u>A.1</u>	
	When the initial conditions for an accident cannot be met, immediate action must be taken to preclude the occurrence of an accident. With the Spent Fuel Pool at less than the required level, the movement of fuel assemblies in the Spent Fuel Pool is immediately suspended. This effectively precludes the occurrence of a fuel handling accident. In such a case, unit procedures control the movement of other (non cask) loads over the spent fuel. This does not preclude movement of a fuel assembly to a safe position.	

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BASES				
ACTIONS (continued)	A.2 When the initial conditions for an accident cannot be met, immediate action must be taken to preclude the occurrence of an accident. With the Spent Fuel Pool at less than the required level, movement of a cask over the Spent Fuel Pool is immediately suspended. This effectively precludes the occurrence of a cask drop accident. In such a case, unit procedures control the movement of other (non cask) loads over the spent fuel. This			
SURVEILLANCE REQUIREMENTS	SR 3 This S event Spent appro level o based Durin equili transf	.7.11.1 SR verifies that sufficient Spent Fuel Pool water is available in the of a fuel handling or cask drop accident. The water level in the t Fuel Pool must be checked periodically. The 7 day Frequency is opriate because the volume in the pool is normally stable. Water changes are controlled by unit procedures and are acceptable, d on operating experience. g refueling operations, the level in the Spent Fuel Pool is at brium with that in the fuel transfer canal, and the level in the fuel fer canal is checked daily in accordance with SR 3.9.6.1.		
REFERENCES	1. 2. 3. 4. 5. 6. 7.	UFSAR, Section 9.1.2. UFSAR, Section 9.1.3. UFSAR, Section 15.11.2. Regulatory Guide 1.25. 10 CFR 100.11. WCAP-7828, December 1971. 10 CFR 50.36		

OCONEE UNITS 1, 2, & 3

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Bases Revision Dated 10/17/01

Attachment 2

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B 3.7 PLANT SYSTEMS

B 3.7.11 Spent Fuel Pool Water Level

BASES

BACKGROUND The minimum water level in the Spent Fuel Pool is consistent with the assumption of iodine decontamination factors following a fuel handling or cask drop accident. <u>The specified water level shields and minimizes the general area dose when the storage racks are filled to their maximum capacity.</u> The water also provides shielding during the movement of spent fuel.

A general description of the Spent Fuel Pool design is given in the UFSAR, Section 9.1.2, Reference 1. The Spent Fuel Pool Cooling and Cleanup System is given in the UFSAR, Section 9.1.3 (Ref. 2). The assumptions of the fuel handling accident or cask drop are given in the UFSAR, Section 15.11.2 (Ref. 3).

APPLICABLE During movement of irradiated fuel assemblies or crane operations with SAFETY ANALYSES loads in the Spent Fuel Pool, the water level in the pool is an initial condition design parameter in the analysis of the fuel handling accident and cask drop accidents in the fuel pool. A minimum water level of 23 ft (Regulatory Position C.1.c of Ref. 4) allows a decontamination factor (DF) of 100 (Regulatory Position C.1.g of Ref. 4) to be used in the accident analysis for iodine. This relates to the assumption that 99% of the total iodine released from the pellet to cladding gap of all the damaged fuel assembly(ies) rods is retained by the Spent Fuel Pool water. The fuel pellet to cladding gap is assumed to contain 10% of the total fuel rod iodine inventory (Ref. 4).

> The fuel handling accident and cask drop accident analysis in the Spent Fuel Pool is described in Reference 3. Since the minimum water level of 21.34 feet is less than 23 feet, the assumed iodine DF must be less than 100, according to Ref. 4, and calculated with comparable conservatism. **Oconee's analysis assumes the top of the irradiated fuel assemblies as the top of the fuel pins (Refs. 4 and 8).** -An experimental test program described in WCAP-7828 (Ref. 6) evaluated the extent of removal of iodine released from a damaged irradiated fuel assembly. Using the analytical results from the test program described in WCAP-7828, with a water depth of 21.34 feet, a comparable DF of 89 was determined. With a minimum water level of

B 3.7.11-1Amondmont Nos. 300, 300, & 300Bases Revision Dated

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APPLICABLE SAFETY ANALYSES (continued)	21.34 ft, and a minimum decay time of 72 hours prior to fuel handling, the analysis and test programs demonstrate that the iodine release due to a postulated fuel handling or cask drop accident is adequately captured by the water, and offsite doses are maintained within allowable limits (Ref. 7).
	The Spent Fuel Pool water level satisfies Criterion 2 and 3 of 10 CFR 50.36 (Ref. 7).
LCO	The specified water level preserves the assumptions of the fuel handling and cask drop accident analyses (Ref. 3). As such, it is the minimum required for fuel storage and movement within the Spent Fuel Pool or movement of the cask over the Spent Fuel Pool.
APPLICABILITY	This LCO applies during movement of irradiated fuel assemblies in the Spent Fuel Pool or movement of the cask over the Spent Fuel Pool since the potential for a release of fission products exists.
ACTIONS	Required Actions A.1 and A.2 are modified by a Note indicating that LCO 3.0.3 does not apply.
	If moving irradiated fuel assemblies or a cask while in MODE 5 or 6, LCO 3.0.3 would not specify any action. If moving irradiated fuel assemblies or a cask while in MODES 1, 2, 3, and 4, the fuel or cask movement is independent of reactor operations. Therefore, in either case, inability to suspend movement of irradiated fuel assemblies or a cask is not sufficient reason to require a reactor shutdown.
	<u>A.1</u>
	When the initial conditions for an accident cannot be met, immediate action must be taken to preclude the occurrence of an accident. With the Spent Fuel Pool at less than the required level, the movement of fuel assemblies in the Spent Fuel Pool is immediately suspended. This effectively precludes the occurrence of a fuel handling accident. In such a case, unit procedures control the movement of other (non cask) loads over the spent fuel. This does not preclude movement of a fuel assembly to a safe position.

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BASES			_		
ACTIONS (continued)	<u>A.2</u>	<u>A.2</u>			
	Whe actio Sper the S the o contr does	n the initial conditions for an accident cannot be met, immediate n must be taken to preclude the occurrence of an accident. With the it Fuel Pool at less than the required level, movement of a cask over Spent Fuel Pool is immediately suspended. This effectively preclude occurrence of a cask drop accident. In such a case, unit procedures rol the movement of other (non cask) loads over the spent fuel. This is not preclude movement of a cask to a safe position.	S		
SURVEILLANCE REQUIREMENTS	<u>SR 3.7.11.1</u>				
	This even Spen appro level base	This SR verifies that sufficient Spent Fuel Pool water is available in the event of a fuel handling or cask drop accident. The water level in the Spent Fuel Pool must be checked periodically. The 7 day Frequency is appropriate because the volume in the pool is normally stable. Water level changes are controlled by unit procedures and are acceptable, based on operating experience.			
	Durir equil trans	ng refueling operations, the level in the Spent Fuel Pool is at ibrium with that in the fuel transfer canal, and the level in the fuel fer canal is checked daily in accordance with SR 3.9.6.1.			
REFERENCES	1.	UFSAR, Section 9.1.2.	—		
	2.	UFSAR, Section 9.1.3.			
	3.	UFSAR, Section 15.11.2.			
	4.	Regulatory Guide 1.25.			
	5.	10 CFR 100.11.			
	6.	WCAP-7828, December 1971.			
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