

February 12, 1988

DO NOT REMOVE

Dockets Nos. 50-259(260)296

Posted  
Amdt. 140  
to DPR-52

Mr. S. A. White  
Manager of Nuclear Power  
Tennessee Valley Authority  
6N 38A Lookout Place  
1101 Market Street  
Chattanooga, Tennessee 37402-2801

Dear Mr. White:

SUBJECT: TECHNICAL SPECIFICATION CHANGE ON ECCS LOGIC INTERVALS TEST  
(TAC 59665/66/67) (TS 212)

Re: Browns Ferry Nuclear Plant, Units 1, 2, and 3

The Commission has issued the enclosed Amendments Nos. 144, 140, and 115 to Facility Operating Licenses Nos. DPR-33, DPR-52 and DPR-68 for the Browns Ferry Nuclear Plant, Units 1, 2 and 3, respectively. These amendments are in response to your application dated August 28, 1985, as supplemented April 7, 1986.

The amendments replace functional test frequencies for core spray logic, RCIC (Initiating and Isolation) logic, HPCI (Initiating and Isolation) logic, ADS logic, LPCI (Initiating and Containment Spray) logic, Core Spray Auto Initiation Inhibit and HPCI Auto Initiation Inhibit from once per 6 months to once per 18 months.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by:

Gary G. Zech, Assistant Director  
for Projects  
TVA Projects Division  
Office of Special Projects

Enclosures:

1. Amendment No. 144 to License No. DPR-33
2. Amendment No. 140 to License No. DPR-52
3. Amendment No. 115 to License No. DPR-68
4. Safety Evaluation

cc w/enclosures:  
See next page

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Mr. S. A. White  
Tennessee Valley Authority

Browns Ferry Nuclear Plant  
Units 1, 2, and 3

cc:

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State Department of Public Health  
State Office Building  
Montgomery, Alabama 36130



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

February 12, 1988

Dockets Nos. 50-259/260/296

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The amendments replace functional test frequencies for core spray logic, RCIC (Initiating and Isolation) logic, HPCI (Initiating and Isolation) logic, ADS logic, LPCI (Initiating and Containment Spray) logic, Core Spray Auto Initiation Inhibit and HPCI Auto Initiation Inhibit from once per 6 months to once per 18 months.

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Sincerely,

Gary G. Zech, Assistant Director  
for Projects  
TVA Projects Division  
Office of Special Projects

Enclosures:

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2. Amendment No. 140 to License No. DPR-52
3. Amendment No. 115 to License No. DPR-68
4. Safety Evaluation

cc w/enclosures:  
See next page



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-259

BROWNS FERRY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 144  
License No. DPR-33

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated August 28, 1985 as supplemented April 7, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-33 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 144, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gary G. Zech, Assistant Director  
for Projects  
TVA Projects Division  
Office of Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 12, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 144

FACILITY OPERATING LICENSE NO. DPR-33

DOCKET NO. 50-259

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages\* are provided to maintain document completeness.

REMOVE

3.2/4.2-46

3.2/4.2-47

INSERT

3.2/4.2-46\*

3.2/4.2-47

TABLE 4.2.B (Continued)  
SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE OR CONTROL THE CSCS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration</u>	<u>Instrument Check</u>
Instrument Channel - RHR Pump Discharge Pressure	(1)	once/3 months	none
Instrument Channel - Core Spray Pump Discharge Pressure	(1)	once/3 months	none
Core Spray Sparger to RPV d/p	(1)	once/3 months	once/day
Trip System Bus Power Monitor	once/operating Cycle	N/A	none
Instrument Channel - Condensate Header Low Level (LS-73-56A, B)	(1)	once/3 months	none
Instrument Channel - Suppression Chamber High Level	(1)	once/3 months	none
Instrument Channel - Reactor High Water Level	(1)	once/3 months	once/day
Instrument Channel - RCIC Turbine Steam Line High Flow	(1)	once/3 months	none
Instrument Channel - RCIC Steam Line Space High Temperature	(1)	once/3 months	none

BFN-Unit 1

TABLE 4.2.B (Continued)  
SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE OR CONTROL THE CSCS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration</u>	<u>Instrument Check</u>
Instrument Channel - HPCI Turbine Steam Line High Flow	(1)	once/3 months	none
Instrument Channel - HPCI Steam Line Space High Temperature	(1)	once/3 months	none
Core Spray System Logic	once/18 months	(6)	N/A
RCIC System (Initiating) Logic	once/18 months	N/A	N/A
RCIC System (Isolation) Logic	once/18 months	(6)	N/A
HPCI System (Initiating) Logic	once/18 months	(6)	N/A
HPCI System (Isolation) Logic	once/18 months	(6)	N/A
ADS Logic	once/18 months	(6)	N/A
LPCI (Initiating) Logic	once/18 months	(6)	N/A
LPCI (Containment Spray) Logic	once/18 months	(6)	N/A
Core Spray System Auto Initiation Inhibit (Core Spray Auto Initiation)	once/18 months (7)	N/A	N/A
LPCI Auto Initiation Inhibit (LPCI Auto Initiation)	once/18 months (7)	N/A	N/A

BFN-Unit 1



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-260

BROWNS FERRY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 140  
License No. DPR-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated August 28, 1985 as supplemented April 7, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 140, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gary G. Zech, Assistant Director  
for Projects  
TVA Projects Division  
Office of Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 12, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 140

FACILITY OPERATING LICENSE NO. DPR-52

DOCKET NO. 50-260

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages\* are provided to maintain document completeness.

REMOVE

3.2/4.2-46

3.2/4.2-47

INSERT

3.2/4.2-46\*

3.2/4.2-47

TABLE 4.2.B (Continued)  
SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE OR CONTROL THE CSCS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration</u>	<u>Instrument Check</u>
Instrument Channel RHR Pump Discharge Pressure	(1)	Once/3 months	none
Instrument Channel Core Spray Pump Discharge Pressure	(1)	Once/3 months	none
Core Spray Sparger to RPV d/p	(1)	Once/3 months	once/day
Trip System Bus Power Monitor	once/operating Cycle	N/A	none
Instrument Channel Condensate Header Low Level	(1)	Once/3 months	none
Instrument Channel Suppression Chamber High Level	(1)	Once/3 months	none
Instrument Channel Reactor High Water Level	(1)	Once/3 months	once/day
Instrument Channel RCIC Turbine Steam Line High Flow	(1)	Once/3 months	none
Instrument Channel RCIC Steam Line Space High Temperature	(1)	Once/3 months	none

BFN-Unit 2

TABLE 4.2.B (Continued)  
SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE OR CONTROL THE CSCS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration</u>	<u>Instrument Check</u>
Instrument Channel HPCI Turbine Steam Line High Flow	(1)	Once/3 months	none
Instrument Channel HPCI Steam Line Space High Temperature	(1)	Once/3 months	none
Core Spray System Logic	once/18 months	(6)	N/A
RCIC System (Initiating) Logic	once/18 months	N/A	N/A
RCIC System (Isolation) Logic	once/18 months	(6)	N/A
HPCI System (Initiating) Logic	once/18 months	(6)	N/A
HPCI System (Isolation) Logic	once/18 months	(6)	N/A
ADS Logic	once/18 months	(6)	N/A
LPCI (Initiating) Logic	once/18 months	(6)	N/A
LPCI (Containment Spray) Logic	once/18 months	(6)	N/A
Core Spray System Auto Initiation Inhibit (Core Spray Auto Initiation)	once/18 months (7)	N/A	N/A
LPCI Auto Initiation Inhibit (LPCI Auto Initiation)	once/18 months (7)	N/A	N/A

BFN-Unit 2



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-296

BROWNS FERRY NUCLEAR PLANT, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 115  
License No. DPR-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated August 28, 1985 as supplemented April 7, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-68 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 115, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gary G. Zech, Assistant Director  
for Projects  
TVA Projects Division  
Office of Special Projects

Attachment:  
Charges to the Technical  
Specifications

Date of Issuance: February 12, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 115

FACILITY OPERATING LICENSE NO. DPR-68

DOCKET NO. 50-296

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages\* are provided to maintain document completeness.

REMOVE

3.2/4.2-45

3.2/4.2-46

INSERT

3.2/4.2-45\*

3.2/4.2-46

TABLE 4.2.B (Cont'd)  
SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE OR CONTROL THE CSCS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration</u>	<u>Instrument Check</u>
Instrument Channel - RHR Pump Discharge Pressure	(1)	once/3 months	none
Instrument Channel - Core Spray Pump Discharge Pressure	(1)	once/3 months	none
Core Spray Sparger to RPV d/p	(1)	once/3 months	once/day
Trip System Bus Power Monitor	once/operating Cycle	N/A	none
Instrument Channel - Condensate Header Level (LS-73-56A, B)	(1)	once/3 months	none
Instrument Channel - Suppression Chamber High Level	(1)	once/3 months	none
Instrument Channel - Reactor High Water Level	(1)	once/3 months	once/day
Instrument Channel - RCIC Turbine Steam Line High Flow	(1)	once/3 months	none
Instrument Channel - RCIC Steam Line Space High Temperature	(1)	once/3 months	none

BFN-Unit 3

TABLE 4.2.B (Cont'd)  
SURVEILLANCE REQUIREMENTS FOR INSTRUMENTATION THAT INITIATE OR CONTROL THE CSCS

<u>Function</u>	<u>Functional Test</u>	<u>Calibration</u>	<u>Instrument Check</u>
Instrument Channel - HPCI Turbine Steam Line High Flow	(1)	once/3 months	none
Instrument Channel - HPCI Steam Line Space High Temperature	(1)	once/3 months	none
Core Spray System Logic	once/18 months	(6)	N/A
RCIC System (Initiating) Logic	once/18 months	N/A	N/A
RCIC System (Isolation) Logic	once/18 months	(6)	N/A
HPCI System (Initiating) Logic	once/18 months	(6)	N/A
HPCI System (Isolation) Logic	once/18 months	(6)	N/A
ADS Logic	once/18 months	(6)	N/A
LPCI (Initiating) Logic	once/18 months	(6)	N/A
LPCI (Containment Spray) Logic	once/18 months	(6)	N/A

BFN-Unit 3



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF SPECIAL PROJECTS

SUPPORTING AMENDMENT NO. 144 TO FACILITY OPERATING LICENSE NO. DPR-33

AMENDMENT NO. 140 TO FACILITY OPERATING LICENSE NO. DPR-52

AMENDMENT NO. 115 TO FACILITY OPERATING LICENSE NO. DPR-68

TENNESSEE VALLEY AUTHORITY

BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2 AND 3

DOCKETS NOS. 50-259, 50-260 AND 50-296

1.0 INTRODUCTION

By letters dated August 28, 1985 and April 7, 1986, Tennessee Valley Authority (the licensee) requested a change to the Browns Ferry Nuclear Plant, Units 1, 2, and 3 Technical Specifications (TS) to revise functional test frequencies listed in Table 4.2.B. The change replaces functional test frequencies for Core Spray logic, Reactor Core Isolation Cooling (RCIC) Initiating and Isolating logic, High Pressure Coolant Injection (HPCI) Initiating and Isolation logic, Automatic Depressurization System (ADS) logic, Low Pressure Coolant Injection (LPCI) Initiating and Containment Spray logic, Core Spray Auto Initiation Inhibit, and HPCI Auto Initiation Inhibit from once per 6 months to once per 18 months.

2.0 EVALUATION

Subsequent to the Salem Anticipated Transient Without Scram (ATWS) events of February 22 and 25, 1983, the NRC issued Generic Letter 83-28 requesting that all licensees of operating reactors, applicants for operating licenses, and holders of construction permits respond to generic issues raised by the analyses of these two ATWS events. Item 4.5.3 of this generic letter requested that licensees and applicants review the existing Reactor Protection System (RPS) on-line functional test intervals required by their plant TS. The licensees/applicants were to ensure that current and proposed intervals for such testing are consistent with a goal of achieving high RPS availability considering uncertainties in component failure rates, uncertainties in common mode failure rates, reduced redundancy during testing, operator errors during testing, and component wear caused by the testing.

The NRC determined that the existing surveillance intervals for channel functional tests, channel calculations and logic system functional tests as defined in NUREG-0123, Revision 3, Boiling Water Reactor Standard Technical Specifications (BWRSTS), provided reasonable assurance of compliance and consistency with the above stated goals and objectives for RPS testing.

The Boiling Water Reactor Owners' Group (BWROG) decided to attempt to resolve these issues generically. It commissioned General Electric (GE) to perform generic analyses and apply the generic results to the individual BWR plants. The generic analyses are applicable to a vast majority of plants that have a relay RPS, as well as to the rest of the plants that have solid-state RPS.

Two GE topical reports were issued: (1) NEDC-30844, in which a representative BWR plant is analyzed and a technical basis is provided for ensuring that the current RPS surveillance test intervals meet the recommendations of Generic Letter 83-28, Item 4.5.3, and (2) NEDC-30851P, in which the base case results from NEDC-30844 are used to establish a basis for extending the current RPS surveillance test intervals and allowable outage times. In these reports, reliability analyses with fault tree modelling were used to estimate RPS failure frequency. The acceptance guidelines used by GE for the TS changes is based on net increase in risk. This is the difference between the increase in risk that would result from the TS changes and the decrease in risk that would result from the reduced likelihood of inadvertent scrams or exceeding the Limiting Conditions of Operations. If the net change in risk is determined to be insignificant, the TS changes are considered acceptable.

On the basis of the review of the topical and the NRC contractors' Technical Evaluation Report (TER), the NRC issued Safety Evaluations (SE) on July 15, 1987 for relay RPS and on January 24, 1988 for solid-state RPS. In both SEs, the NRC staff found the analyses presented acceptable for supporting a determination that the BWRSTS RPS test intervals are consistent with the high RPS availability requested by Generic Letter 83-28, Item 4.5.3.

The TS for Browns Ferry Nuclear Plant, Units 1, 2 and 3, currently require functional testing once per 6 months. The licensees' request to extend the functional testing to once per 18 months will allow performance of complex logic surveillance testing at a time when the unit is shut down. These complex surveillance tests require numerous temporary alterations and/or administrative controls and are inherently prone to undesired actuations and potential for operator error while also reducing redundancy of protection during the testing. Testing the RPS system at 18 month intervals is consistent with the interval that the NRC has determined to be adequate to achieve the goal of high RPS availability considering uncertainties in component failure rates, uncertainties in common mode failure rates, reduced redundancy during testing, and component wear caused by the testing. The staff, therefore, finds this change acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed

finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of these amendments.

#### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: John Stang

Dated: February 12, 1988